SNL/CA Collaborations

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Laboratory Studies in Support of Collaborations

Plasma Surface Interactions Science Center

- Plasma driven tritium retention and permeation using the Tritium Plasma Experiment (TPE)
 - Extend permeation experiments to neutron irradiated materials (post-TITAN)
- Sheath power transmission (see talk on Friday morning)
- Characterization of nanostructured tungsten
- Study of early stage growth of nanostructured tungsten
- Compact Torus Injection Experiment (CTIX)
 - PSI studies
 - Development of refractory electrodes











PSI Science Center (Tritium Permeation)

Completed thermal testing of high temperature tritium retention stage for TPE

- Stable operation at 800 °C for 1 hour (1000 °C for shorter times)
- Feedback control of He flow to be implemented to reduce thermal ramp up time (now ~ 30 minutes)
- Design and fabrication of permeation stage in progress
 - Assembly for testing sample sealing designed and being fabricated
 - Final design in progress (evaluating bellows versus o-ring design)
 - He flow gas to capture permeating D/T between cooling fins and sample





PSI Science Center (Tritium Permeation)

- Sensitivity estimated from ion driven permeation flux experiments in HiFIT (Osaka Univ.)
 - $\Gamma_{\text{beam}} = 10^{20} \text{ Dm}^2 \text{s}^{-1} \text{ flux (1 keV)}$
 - $10^{14} \text{ D m}^{-2}\text{s}^{-1} < \Gamma_{p} < 10^{17} \text{ D m}^{-2}\text{s}^{-1}$
- Adjust for TPE
 - Geometry (sample dia. and thickness)
 - Tritium seeded plasma and higher flux
- Expected permeation
 - 2x10⁻¹⁰ Ci s⁻¹ < I_p < 2x10⁻⁷ Ci s⁻¹
- Using 1000 sccm He gas flow in TPE
 - 10^{-5} Ci m⁻³ < A < 10^{-2} Ci m⁻³ (1000 cc ion chamber covers μ C m⁻³ to C m⁻³)
 - Resonance time for this flow would be 60 s
- Breakthrough times up to 10⁴ s observed in HiFIT
 - Upgrade to TPE tritium gas handling would be required (> 4 hour runs)
 - Less important at higher temperatures expected in a divertor







post-TITAN (Tritium Permeation)

- The US-Japan TITAN program will transition to post-TITAN in FY13
 - "Tritium, Irradiation and Thermofluid for American and Nippon"
- post-TITAN
 - "Technological Assessment of Plasma Facing Components for DEMO Reactors"
 - Three Tasks
 - Heat Load Tests, Heat Transfer, and System Design
 - Neutron-irradiation Effects, Microstructure, and Physical Properties
 - Plasma-Surface Interactions and Tritium Behavior (INL / SNL-CA)
- Challenges to study permeation of neutron damaged materials
 - Damage will increase trapping, hence longer breakthrough times
 - Strength of samples may be compromised
 - Neutron irradiations in HFIR may limit sample diameter to 6 mm. Alternative sealing techniques are being considered





PSI Science Center (Nanostructured W)

First GIXRD results of tungsten fuzz (rolled PLANSEE tungsten)



PSI Science Center (Nanostructured W)

 Continuing to develop an experimental capability to observe early stage growth of nanostructured W (Scanning Tunneling Microscopy)



- Planning for STM availability in FY13
- Characterization of compact ECR plasma source in progress
- Heater stage will be used to measure growth rates in FY12





CTIX (PSI Studies and PFC Development)

- Compact Toroid (CT) accelerators can be used for deep fueling of hydrogenic species (CTIX: V_{CT} ~ 200 km/s with n > 10¹⁴ cm⁻³)
- UC Davis and SNL/CA collaboration exploring PSI issues and improvements to PFCs for CTIX



- Gas fueling improvements being implemented
- New electrodes with tungsten coating will address impurity production



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