

Modeling core impurity reduction via divertor gas injection in NSTX

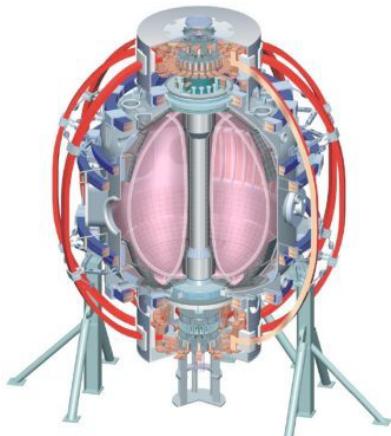
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Plasma Facing Components
PPPL, Princeton, NJ
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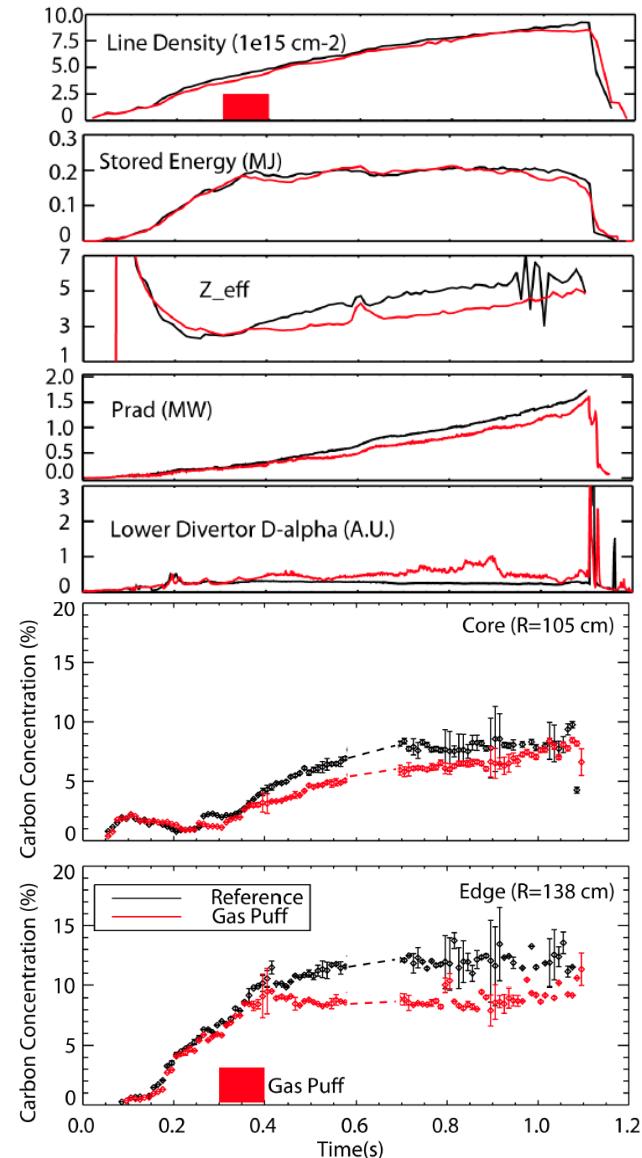
*Culham Sci Ctr
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In NSTX, lithium conditioning led to impurity accumulation

- Lithium conditioning → impurity accumulation
 - Lithium-induced edge stabilization suppresses ELMs, allowing accumulation
- High impurity concentration can be problematic
 - P_{rad} up to 2 MW (largely due to high-Z impurities)
 - Lack of density control
 - Z_{eff} increase → resistivity increase
- Impurity control techniques on NSTX
 - ELM triggering with resonant magnetic perturbations (RMPs) [Canik PRL 2010]
 - Control plasma-wall interaction during startup phase
 - Partially detached divertor scenarios (gas puff, impurity seeding, snowflake)
 - **Deuterium gas puffing [Scotti APS 2010]**

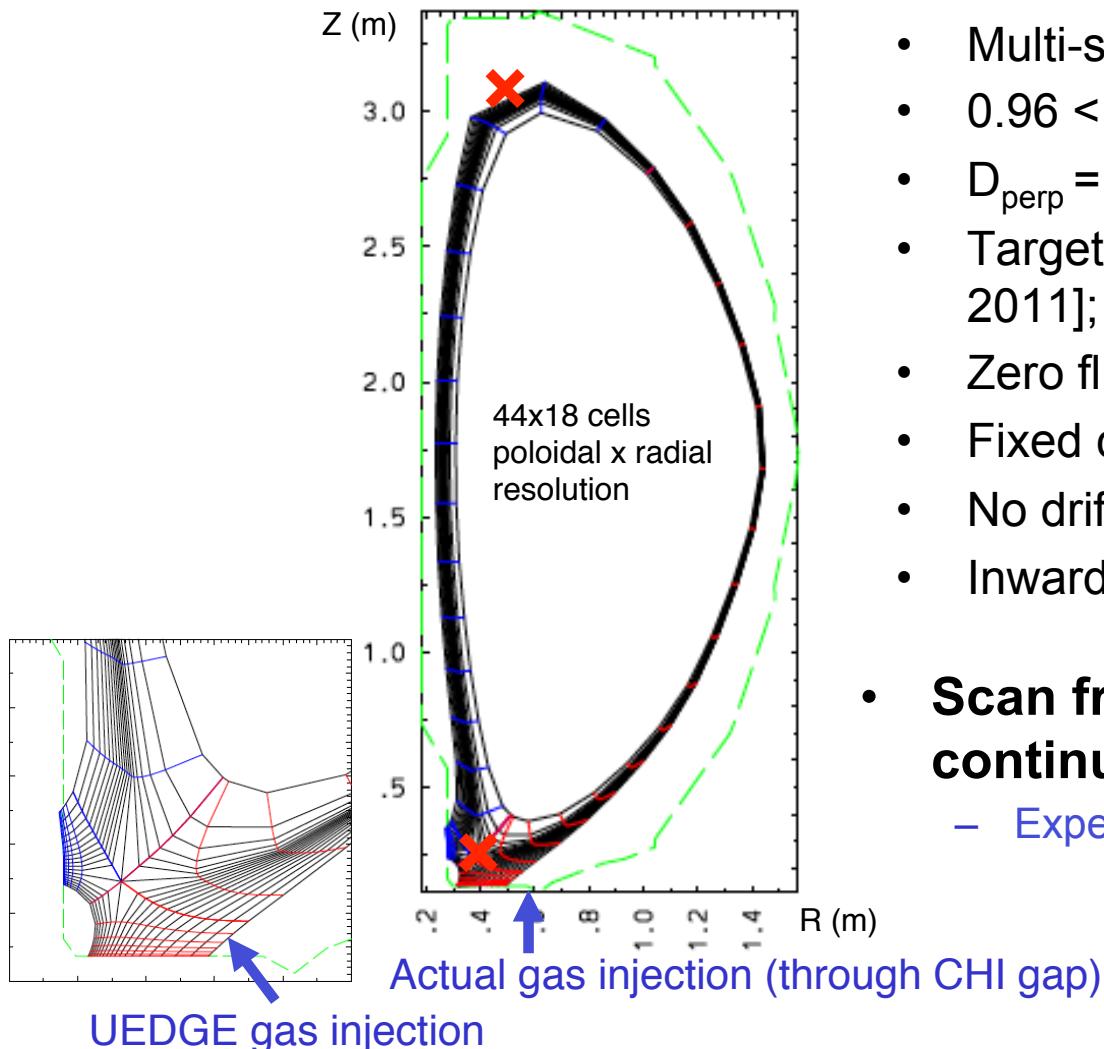
Divertor deuterium puffing on NSTX reduced impurity concentration by up to 30%

- ~20 torr-l injected in 0.1 sec
 - Core plasma retains desirable properties
 - Outer divertor remains attached
 - Carbon concentration reduced 30%
- Deuterium puffing might...
 - Reduce sputtered influx
 - Modify parallel impurity transport
 - Divertor impurity retention
 - Other?



The UEDGE 2D fluid transport code is used to study effects of gas puffing on carbon transport

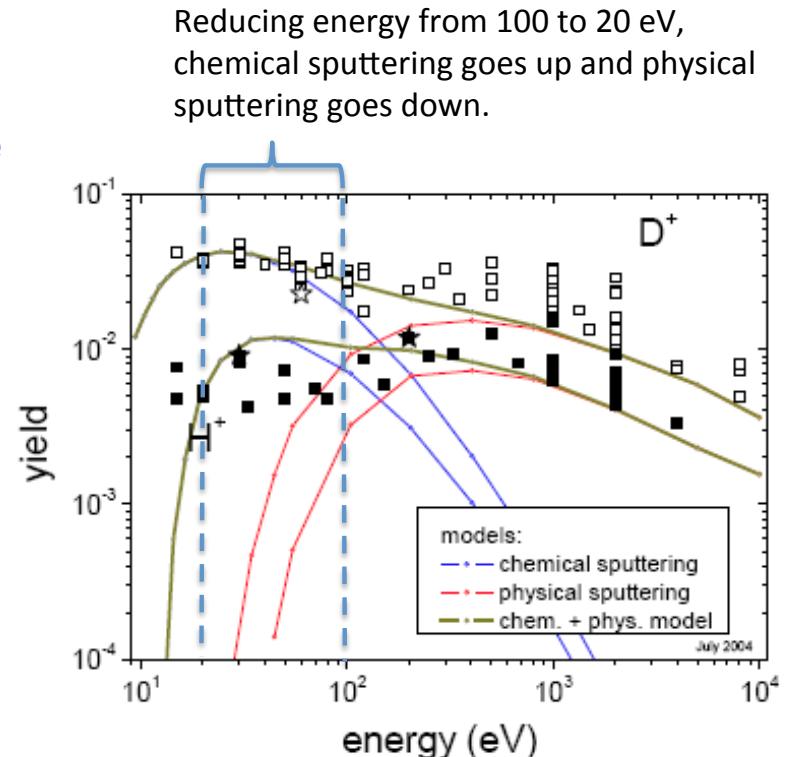
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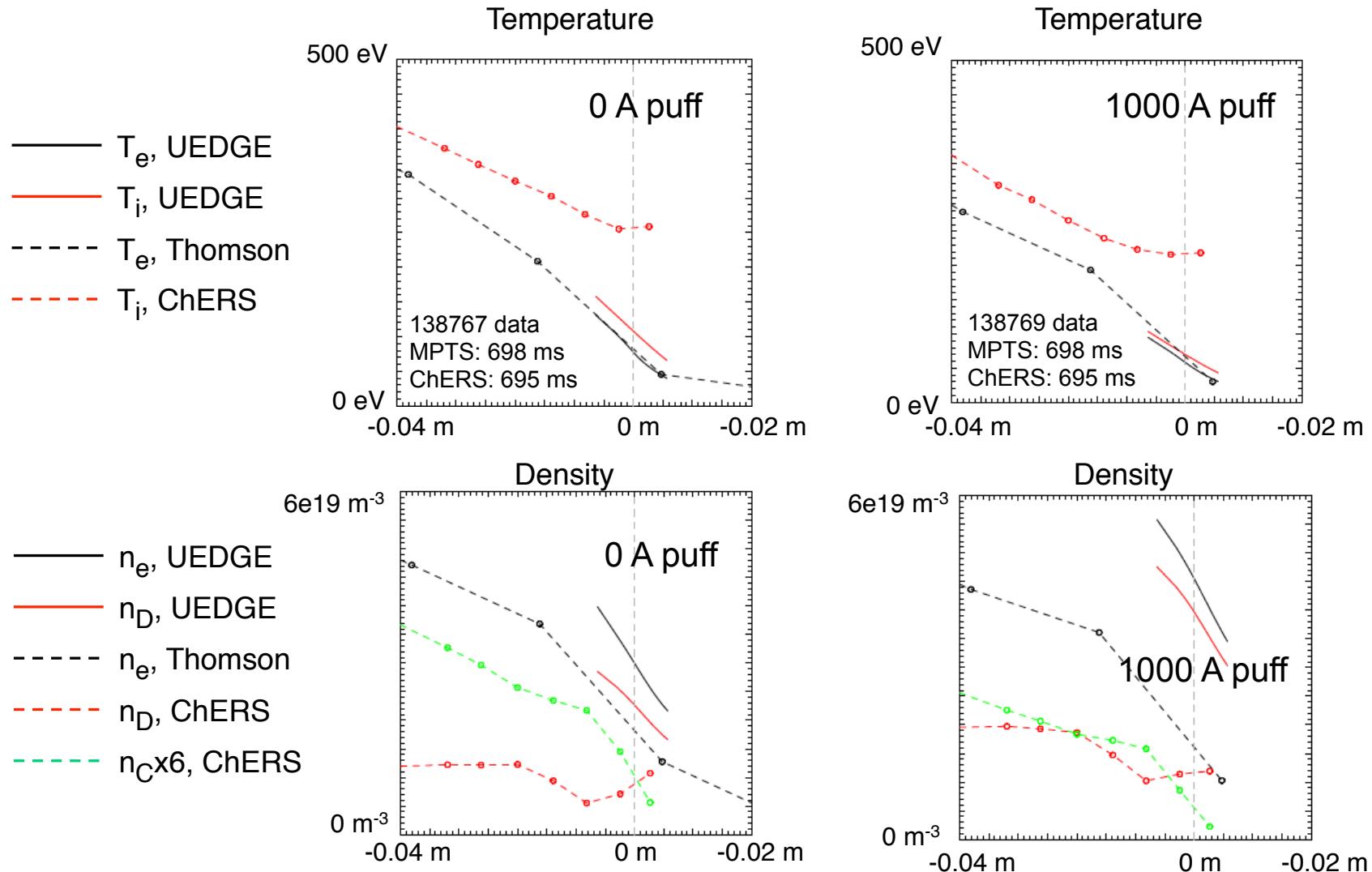
- Multi-species carbon model (C^{1+} - C^{6+})
- $0.96 < \text{psi} < 1.028 \rightarrow \sim 6 \text{ mm SOL}$
- $D_{\text{perp}} = 0.5 \text{ m}^2/\text{s}$, $\chi_{i,e} = 1.5 \text{ m}^2/\text{s}$
- Target recycling is 90% [Canik PoP 2011]; Wall recycling is 100%
- Zero flux BC for neutral D and C at core
- Fixed core flux of D^+
- No drift effects
- Inward carbon pinch, $v_{\text{pinch}} = -25 \text{ m/s}$
- **Scan from 0 to 1200 atom-amps continuous D injection**
 - Experimental rate is 2000 A for 0.1 s

UEDGE includes physical and chemical carbon sputtering

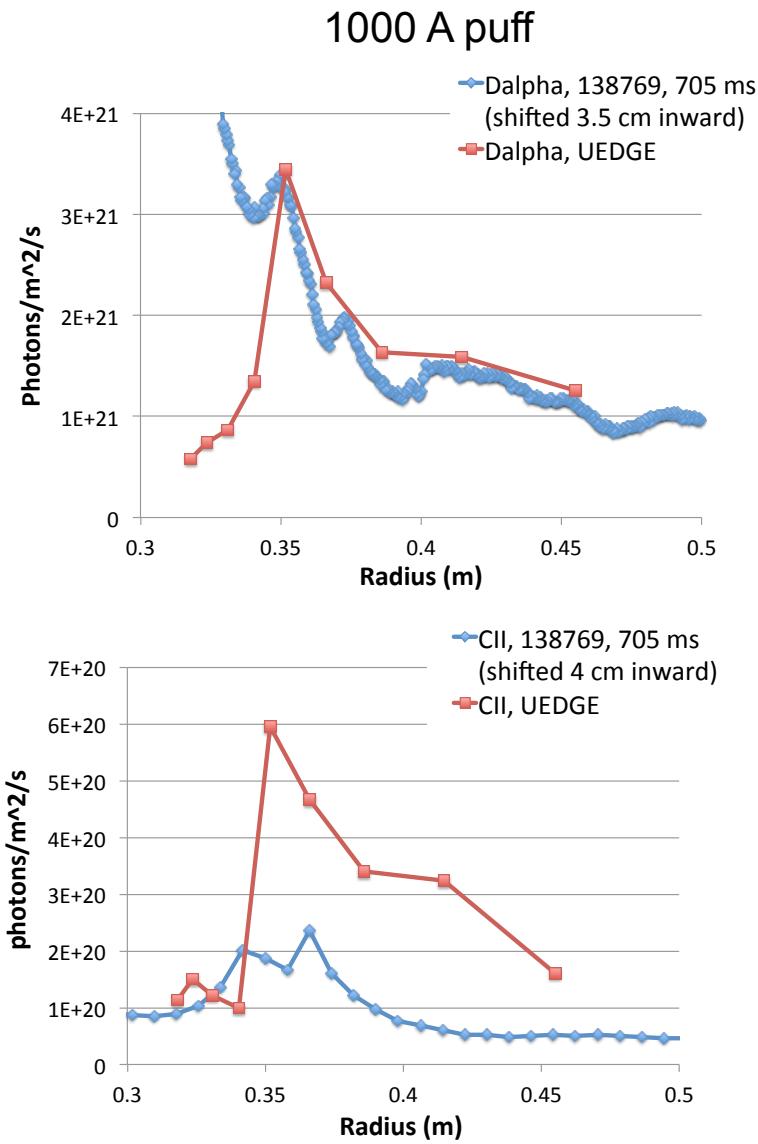
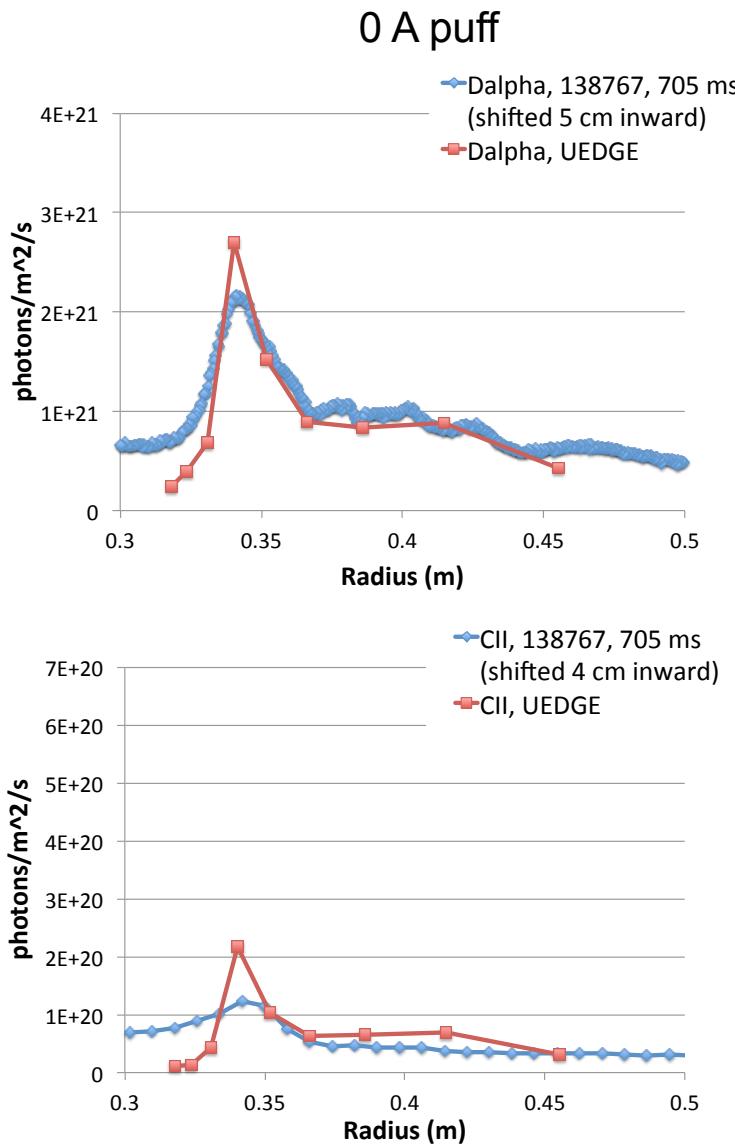
- UEDGE includes physical and chemical sputtering of carbon
 - Physical and chemical sputtering models are from DIVIMP (U. Toronto)
 - Actual NSTX vessel wall is far from outer UEDGE boundary
 - Sputter yield reduced 10x at outer UEDGE boundary
 - $T_{target} = 500$ K and $T_{wall} = 300$ K assumed for all gas puff rates
 - Experimental T_{target} drops from ~600 K to ~400 K
- Lithium coating effects are not modeled
 - Complicated Li-C-D-O interaction still under investigation [e.g., Scotti PSI 2012]
- C-C and Li-C sputtering not included



Midplane T_e matches well; T_i , n_e , n_i , not well-matched

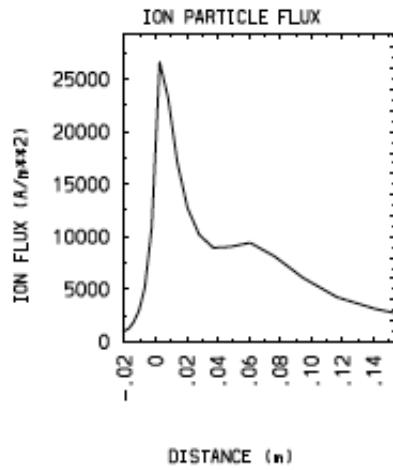
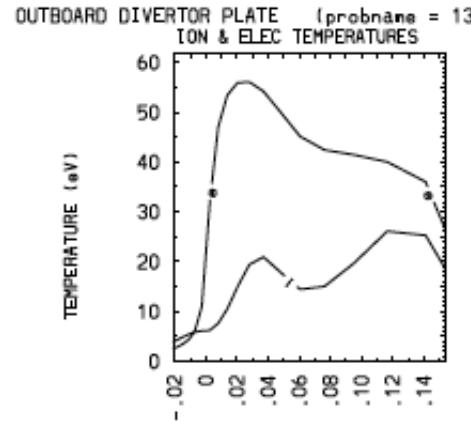
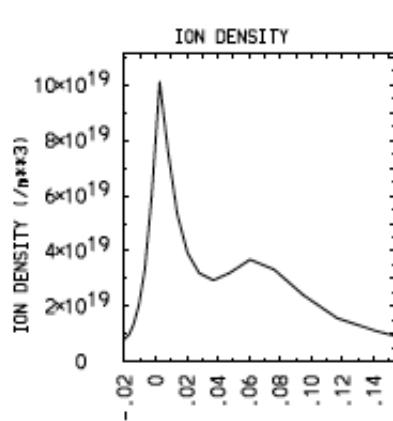


Outer divertor D _{α} profiles show good agreement; UEDGE CII profiles with large injection are too high

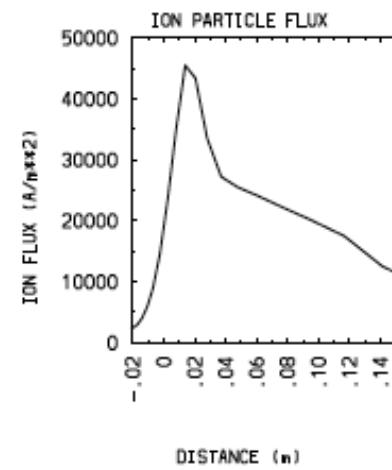
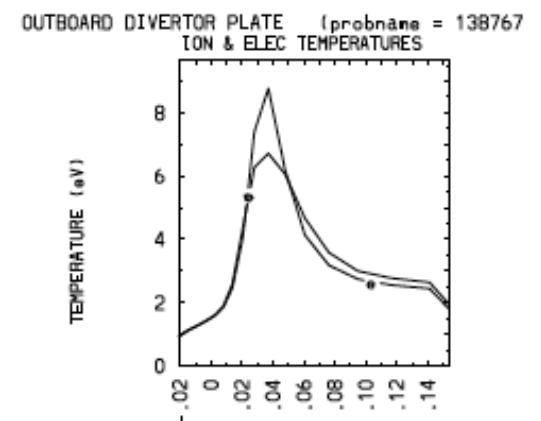
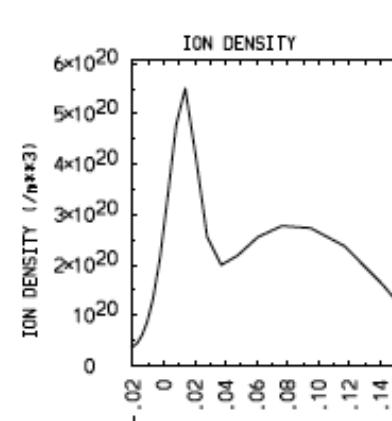


Outer divertor sees large temperature and heat flux reduction with 1000 A injection

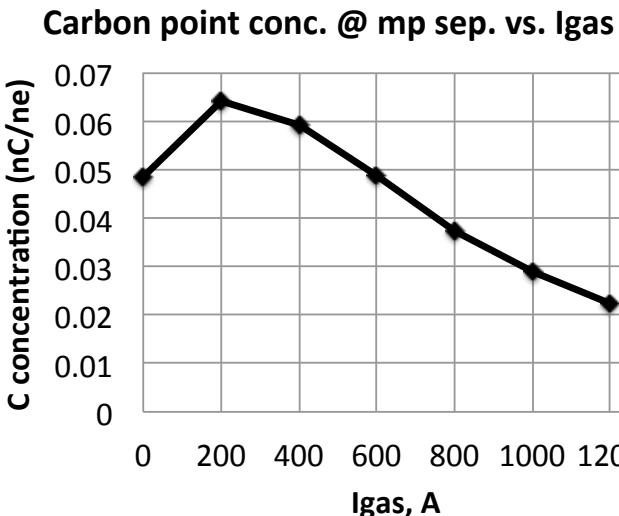
No injection



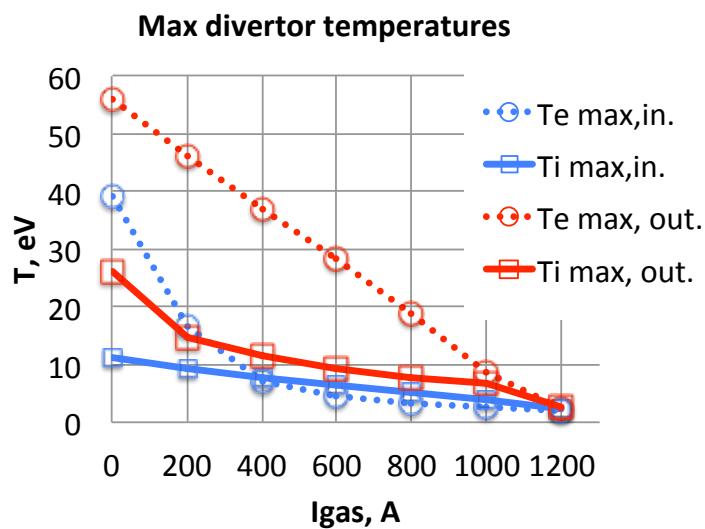
1000 A injection



Carbon concentration is reduced with increasing deuterium gas injection

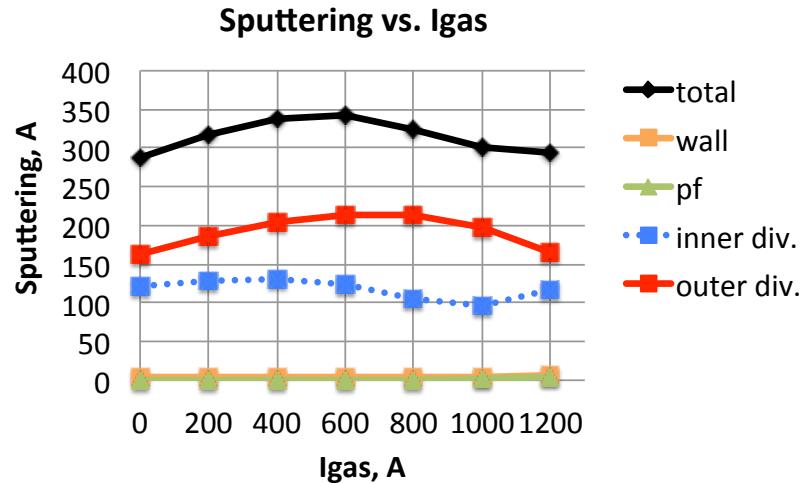


- Carbon concentration at the midplane separatrix is reduced by over 50% with 1200 A puff

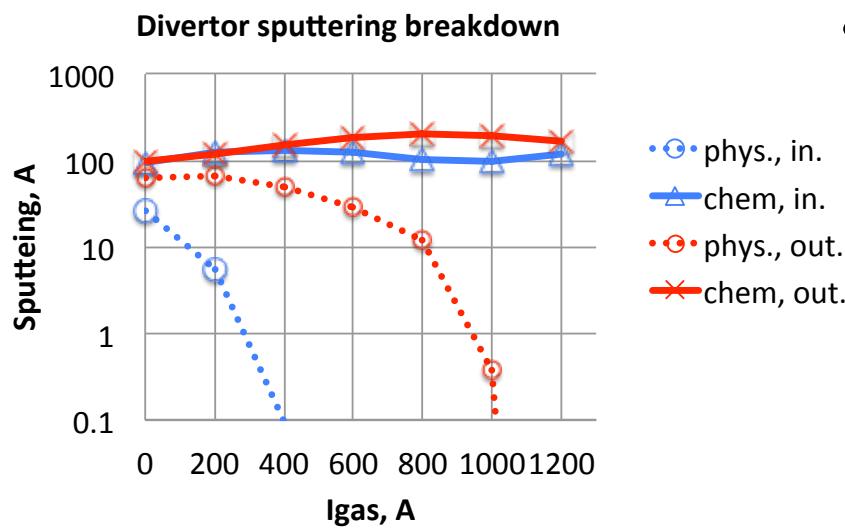


- Divertor temperatures are reduced dramatically

Total sputtered flux remains nearly constant

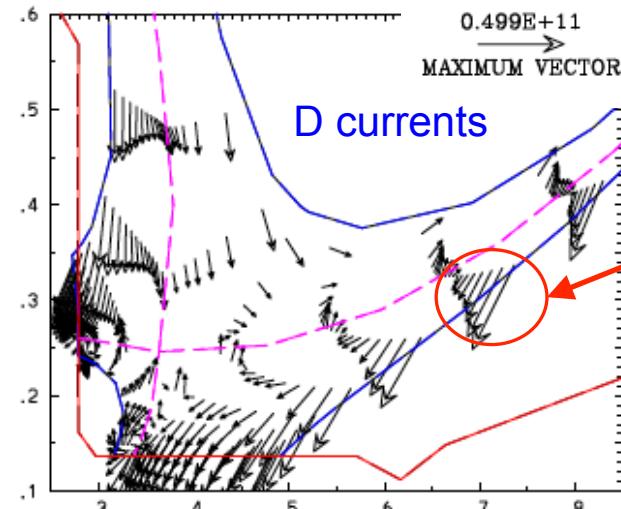


- Sputtered flux is dominated by inner and outer divertor sources



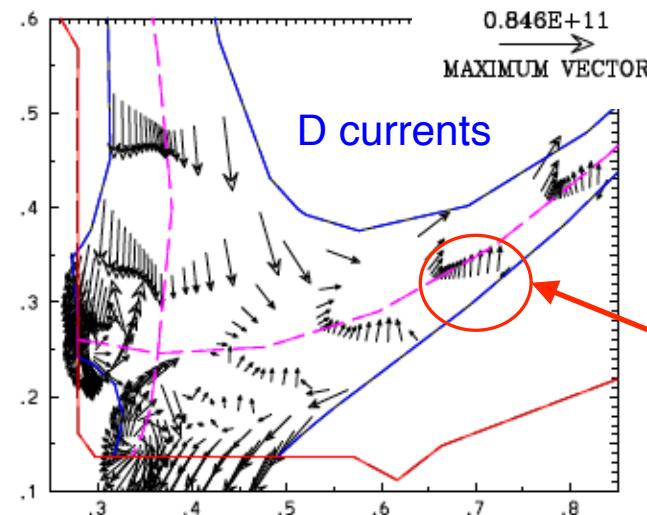
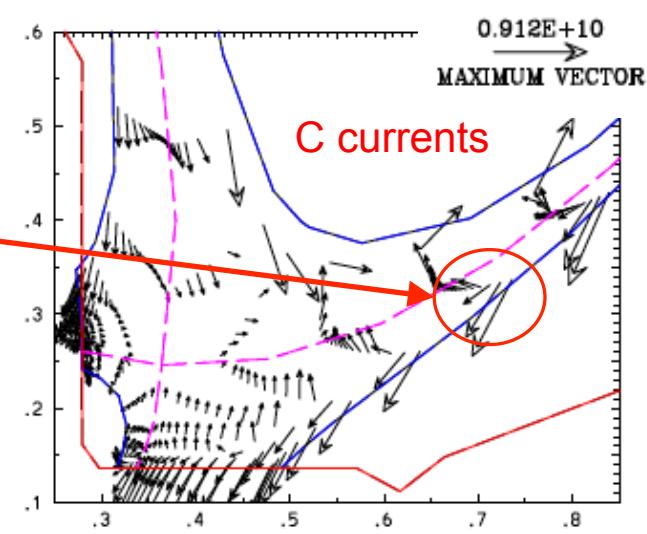
- Physical sputtered flux drops, but chemical sputtered flux rises

D gas injection causes D and C flow away from outer divertor



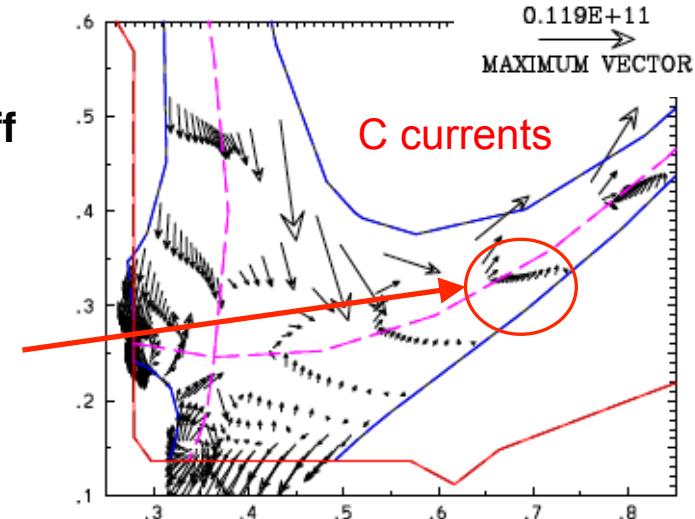
Flow toward
outer divertor

0 A puff



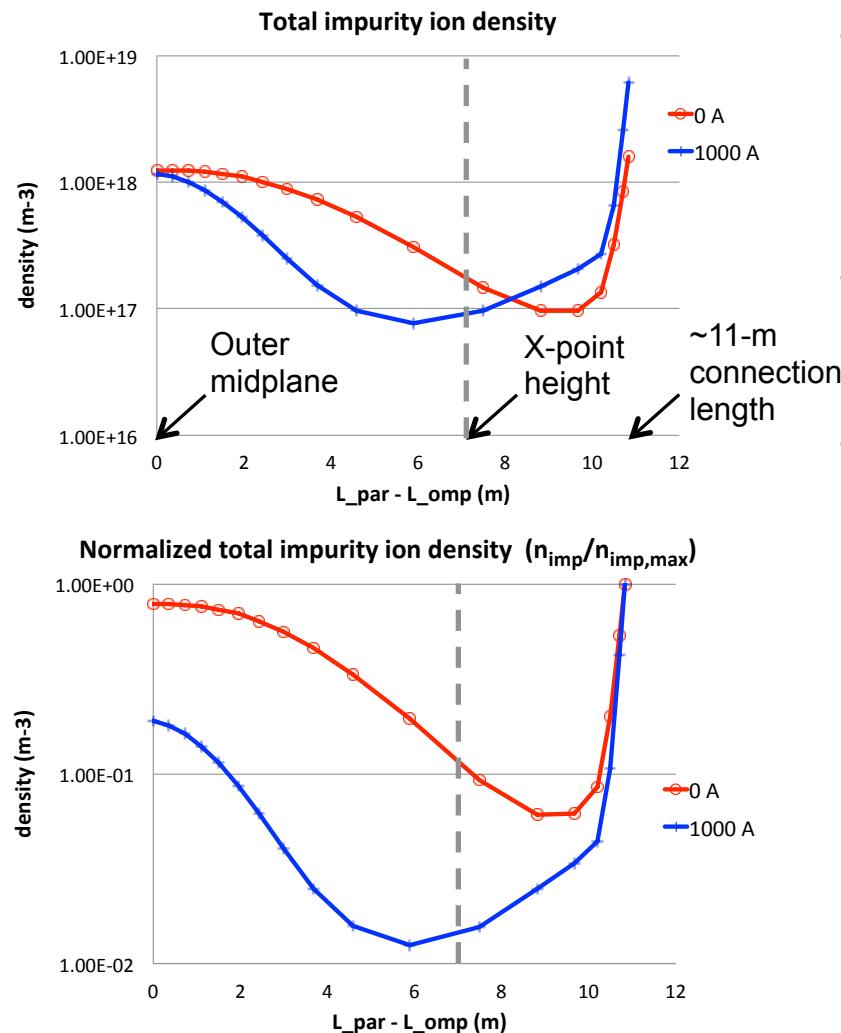
1000 A puff

Flow away from
outer divertor



Total impurity density near the X-point is reduced with deuterium gas injection

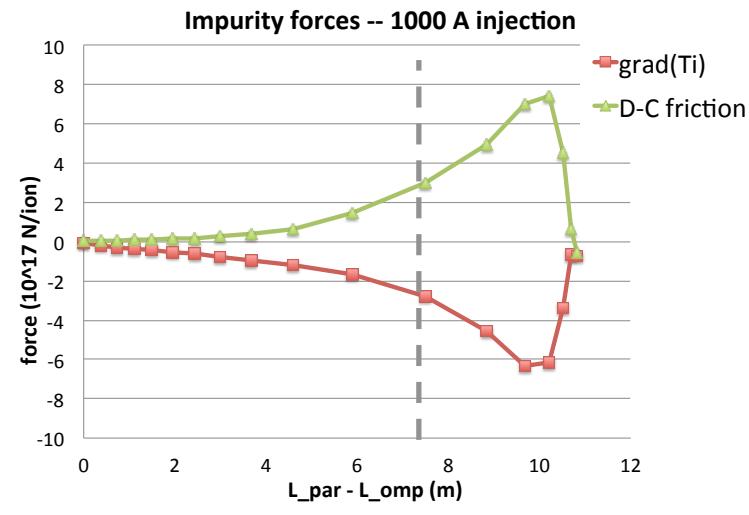
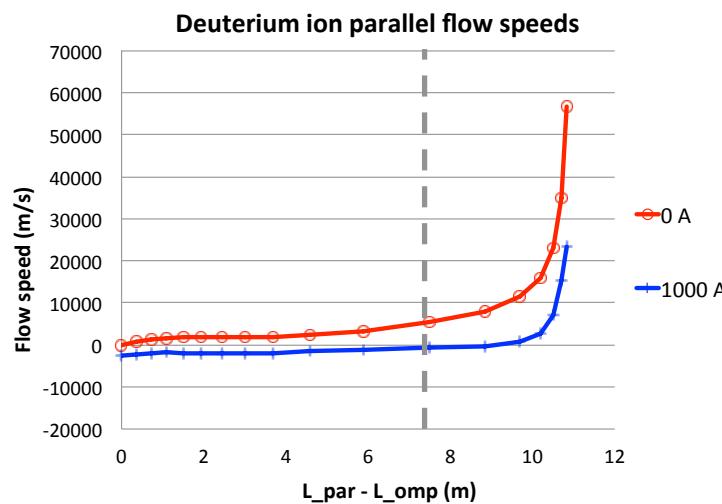
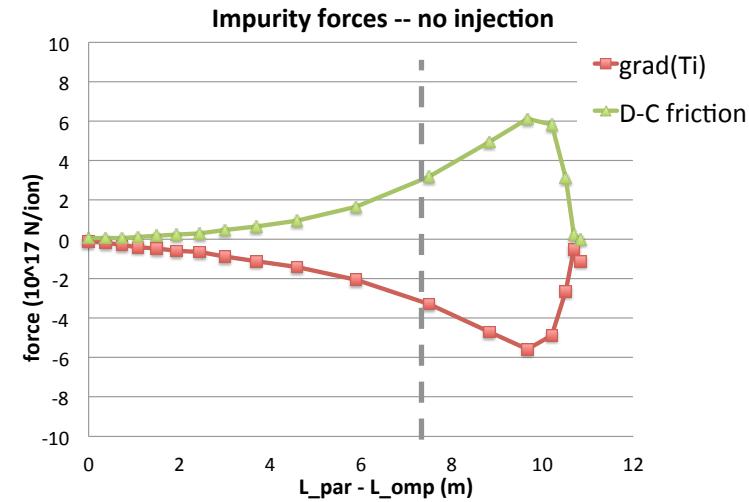
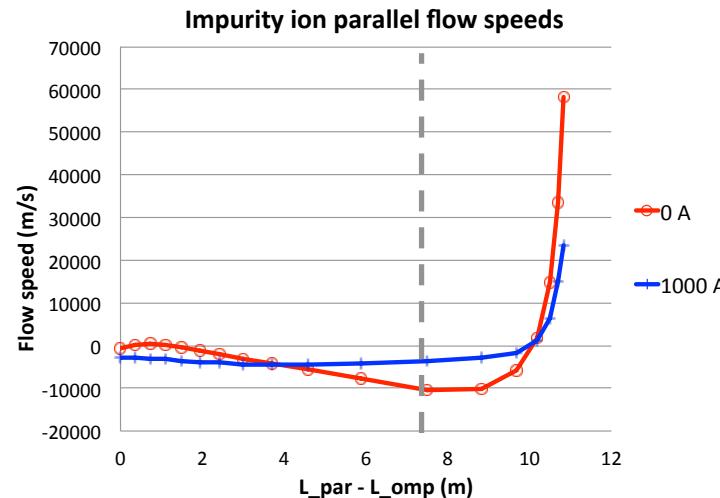
Data is plotted along a field line on the 2 mm flux surface.



- n_{imp} rises at outer target, but falls (slightly) at outer midplane
- n_{imp} at X-point height is reduced
- $n_{\text{imp}}/n_{\text{imp,max}}$ shows large relative reduction
 - Divertor retention?

Deuterium gas injection prevents carbon flow stagnation near outer midplane

Data is plotted along a field line on the 2 mm flux surface.



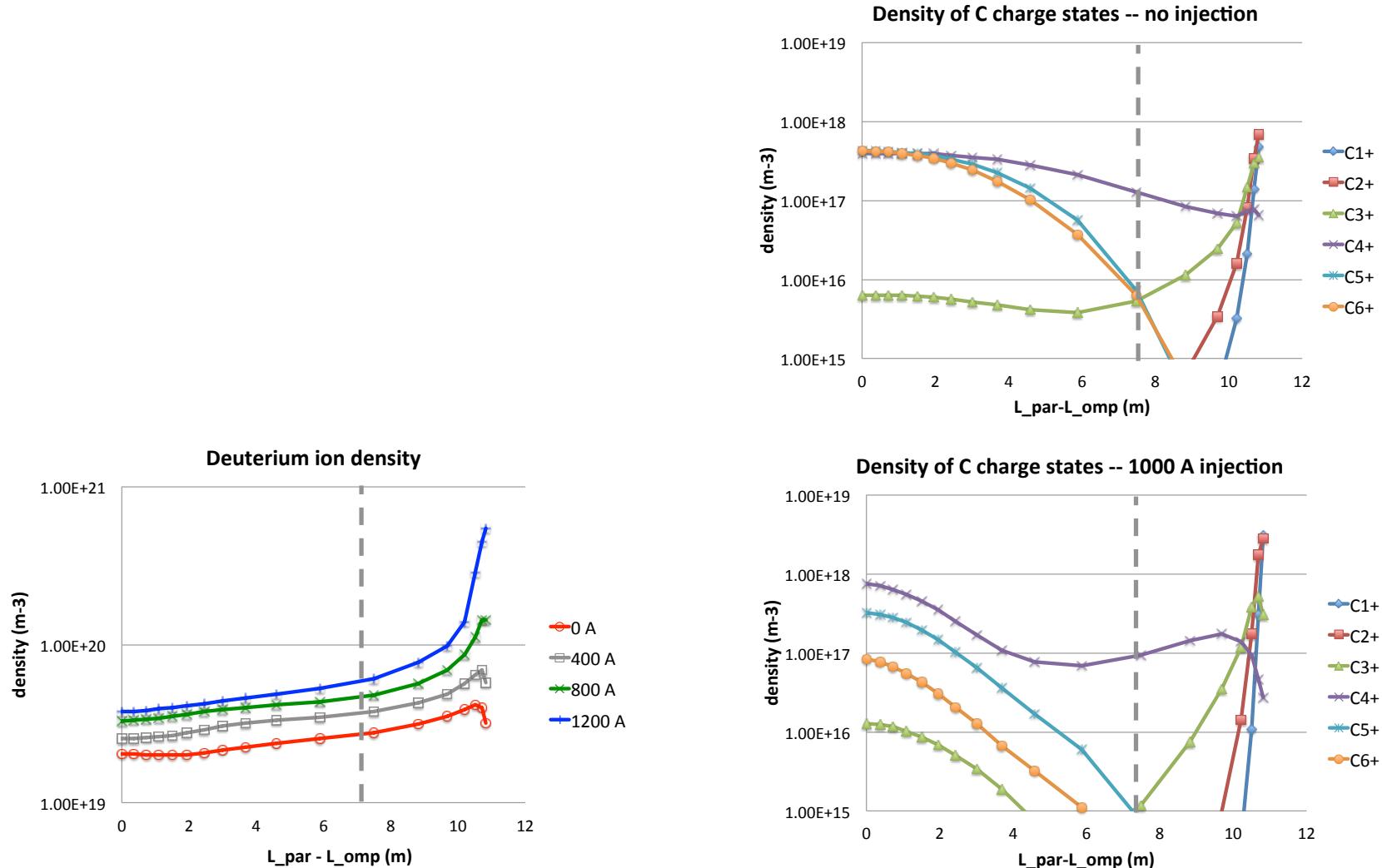
Conclusions

- UEDGE gas puff study shows carbon impurity reduction with divertor deuterium gas injection
 - Observed reduction trend is consistent with experiment
 - Reduction seems related to “flow-through” of C past outer midplane
 - Carbon buildup at outer midplane is prevented
 - Reduction of carbon source is not seen
- Simulations could be improved in future work
 - Double null grid would give larger SOL and include upper divertor physics
 - More closely matching midplane profiles and divertor spectroscopy would increase confidence

Backup slides

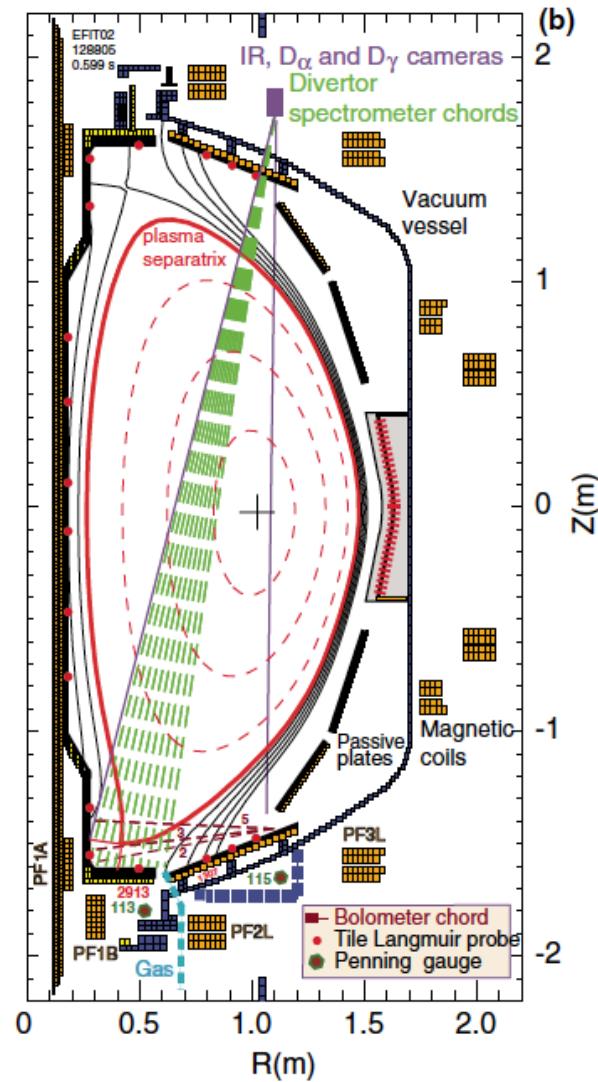
D and C profiles...

(Abscissa is the distance from the outer midplane along a field line on the 2 mm flux surface.)



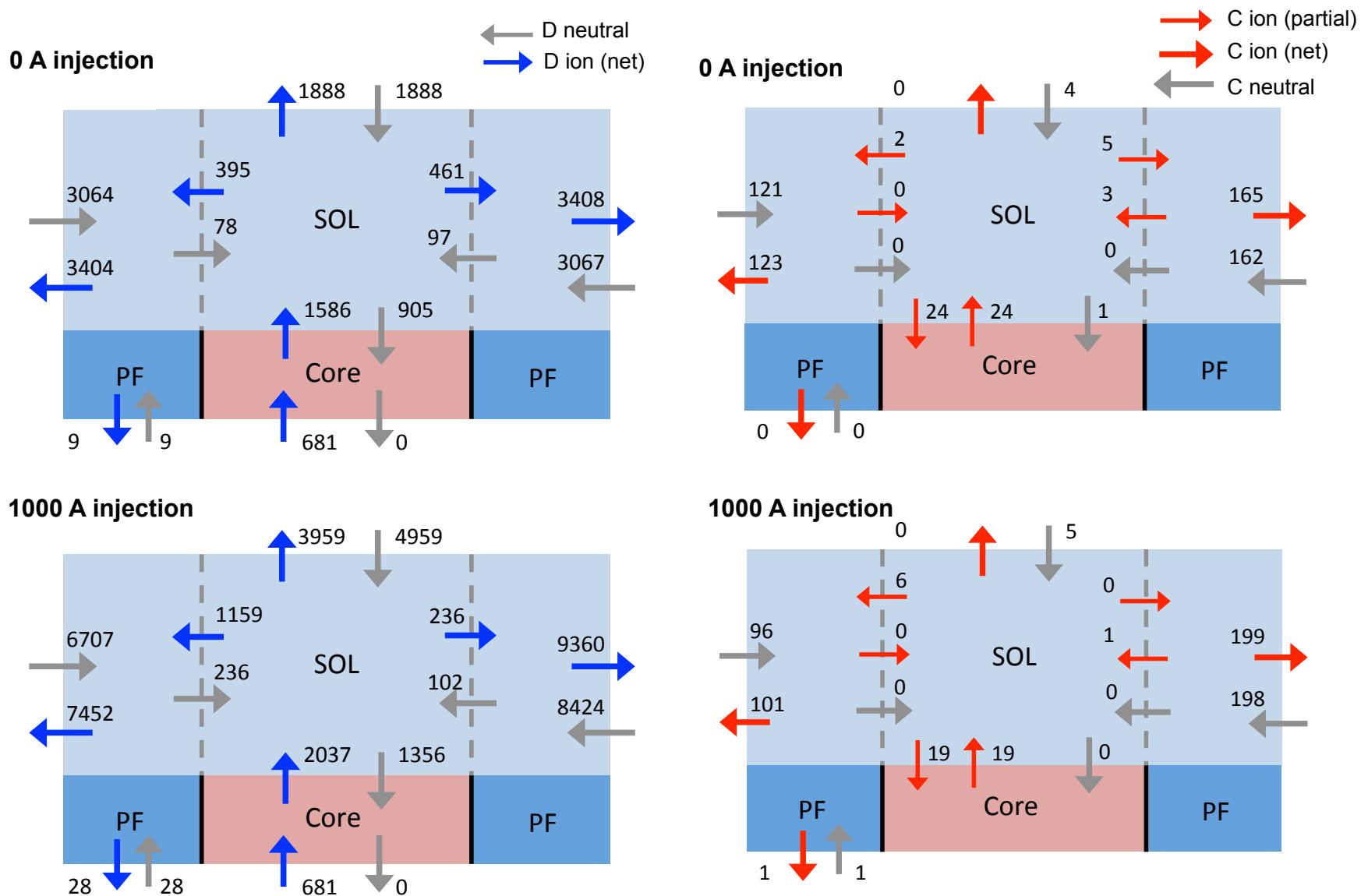
Divertor diagnostics include IR and visible cameras, divertor spectrometry, and tile Langmuir probes

- IR camera used to determine heat flux
- Visible cameras can be filtered to provide D_α data
- Langmuir probes provide sparse data
 - Other comments???



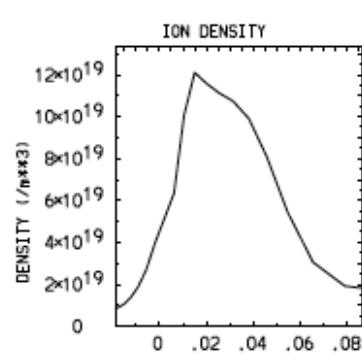
[Soukhanovskii NF '09]

With deuterium injection, recycling increases dramatically, but changes to C transport are more subtle

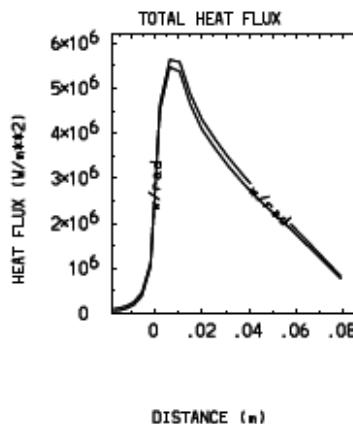
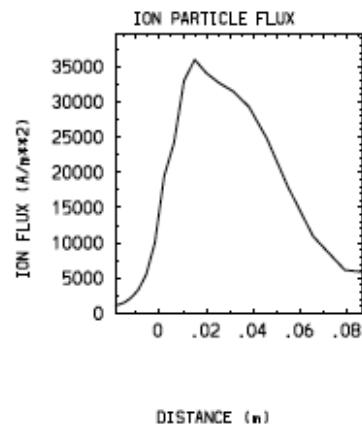
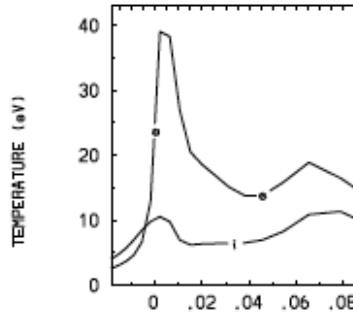


With 1000 A injection, inner divertor temperature and heat flux fall dramatically

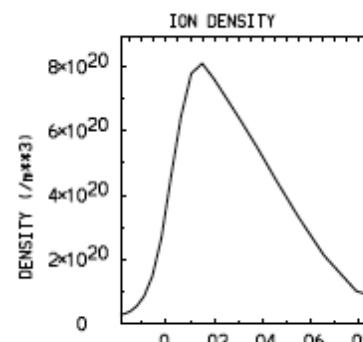
0 A injection



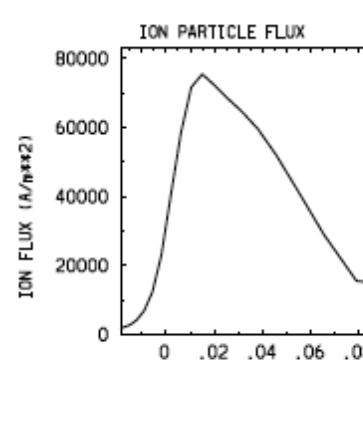
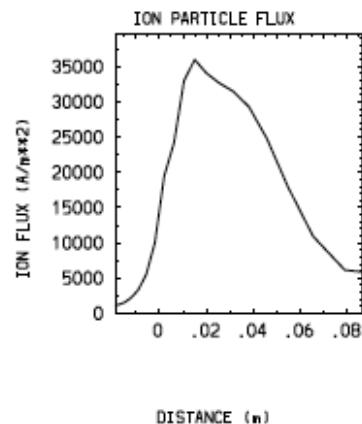
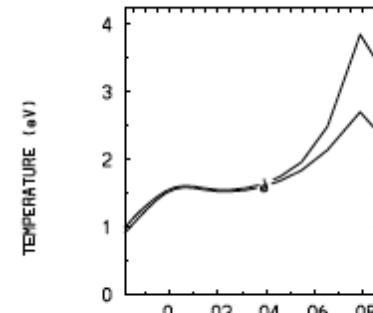
INBOARD DIVERTOR PLATE (propane = 138767)
ION & ELEC TEMPERATURES



1000 A injection

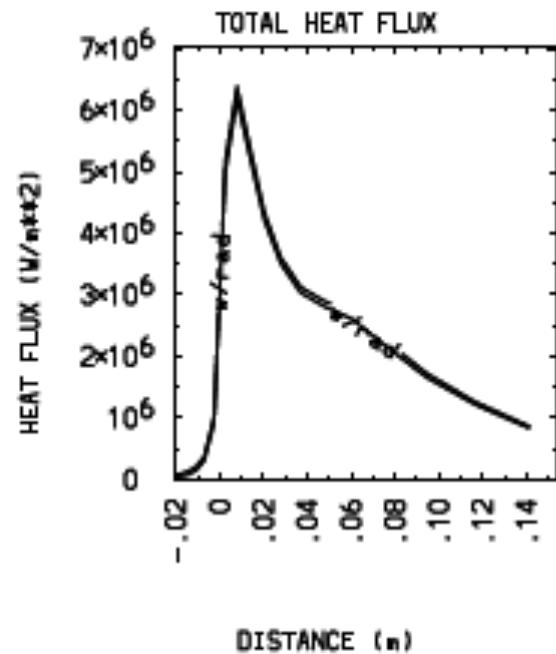


INBOARD DIVERTOR PLATE (propane = 138767)
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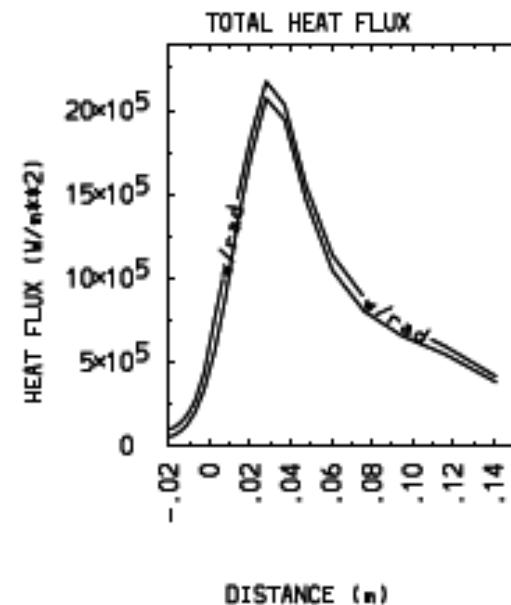


Heat flux comparison...

0 A puff



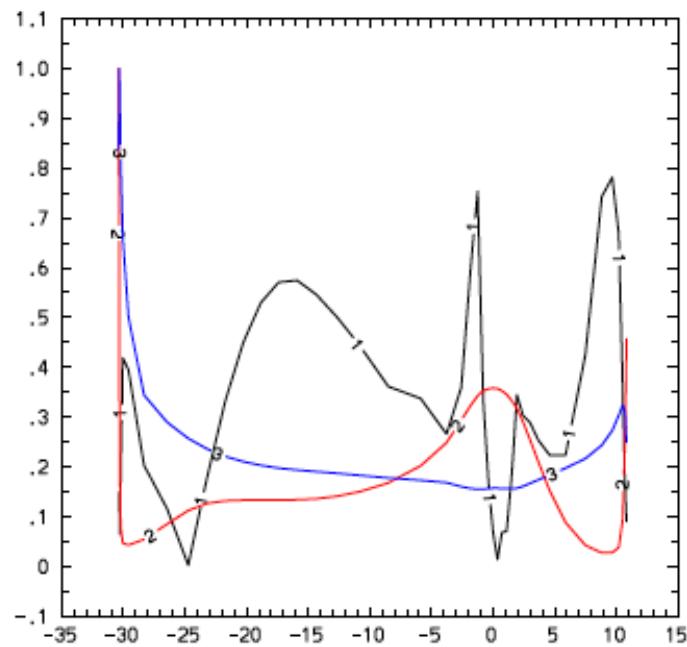
1000 A puff



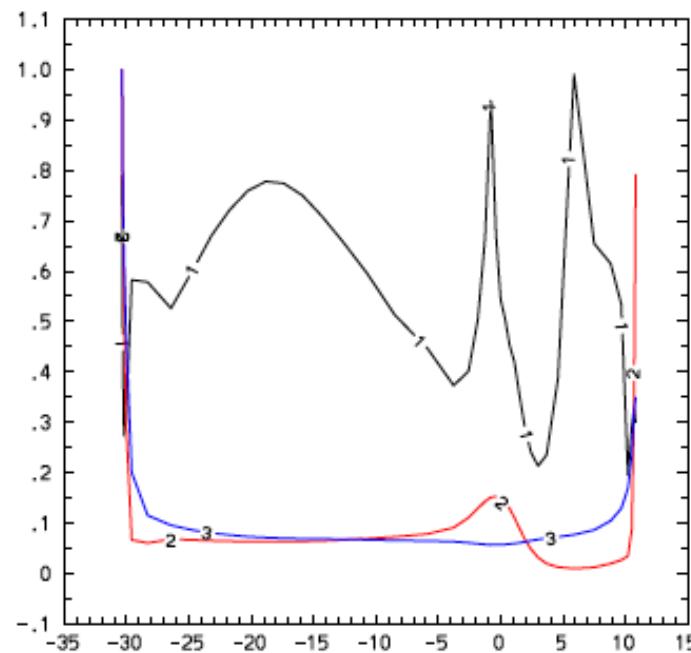
I _{gas} (A)	P _{SOL} (MW)	P _{div_in} (MW)	P _{div_out} (MW)	P _{wall} (MW)	P _{rad}
0	3	0.44	0.95	1.51	0.22
1000	3	0.27	0.37	1.92	0.50

Assessment of radial impurity current

0 A



1000 A



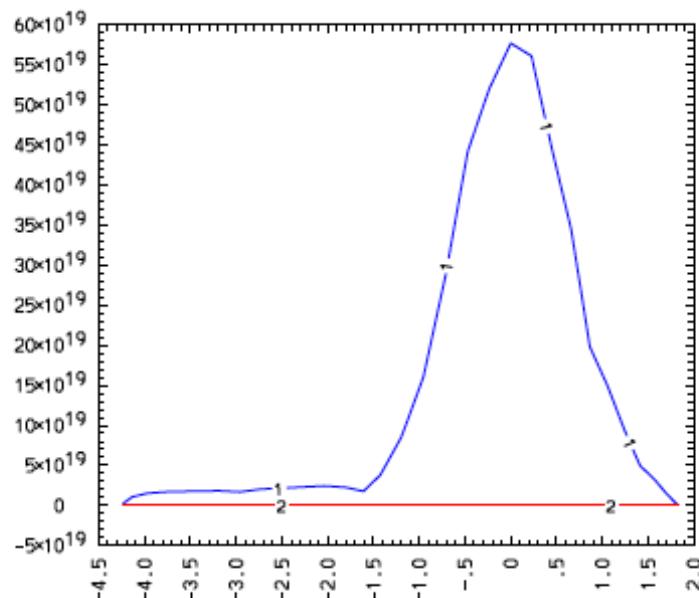
1. (Black) Poloidal impurity current (into a cell) / total current (into a cell)
2. (Red) Impurity ion density
3. (Blue) Deuterium ion density

Abscissa is poloidal distance from outer midplane. Data is plotted for the 2 mm flux surface ($iysptrx+6$) are plotted.

```
1: plot ipc(1:nx)/itc(1:nx) ltmp(1:nx,iy)-ltmp0  
2: plot nimp(1:nx)/max(nimp(1:nx)) ltmp(1:nx,iy)-ltmp0 color=red  
3: plot ni(1:nx,iy,1)/max(ni(1:nx,iy,1)) ltmp(1:nx,iy)-ltmp0 color=blue
```

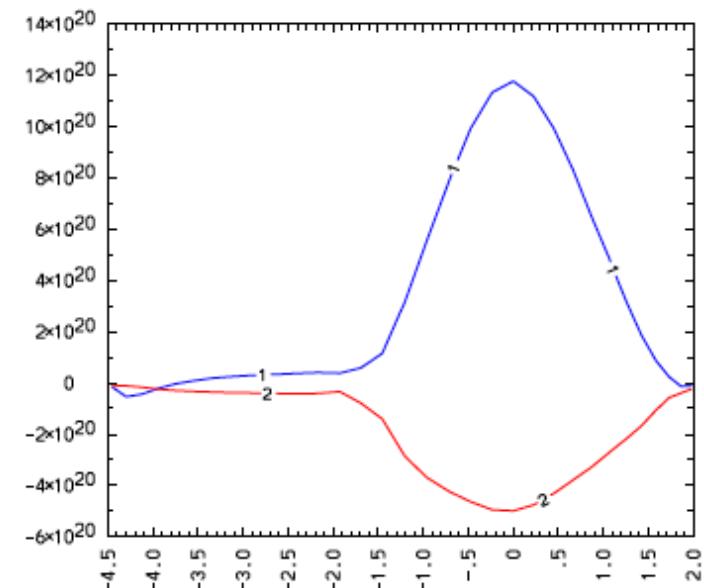
Radial deuterium fluxes, 0 A

Core flux



```
1: plot fniy(ixpt1+1+ixpt2,0,1) lpol(ixpt1+1+ixpt2,0)-lpol0 color=blue  
2: plot fngy(ixpt1+1+ixpt2,0,1) lpol(ixpt1+1+ixpt2,0)-lpol0 color=red
```

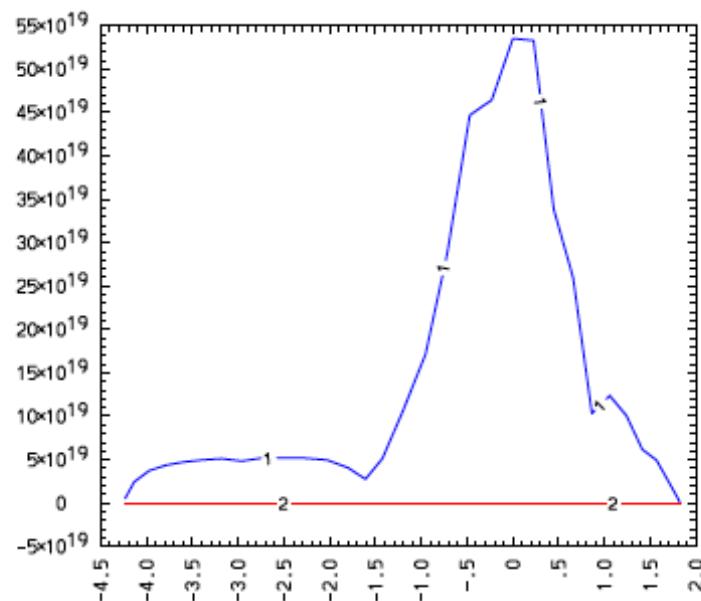
Separatrix flux



```
1: plot fniy(ixpt1+1+ixpt2,1ysptrx,1) lpol(ixpt1+1+ixpt2,1ysptrx)-lpol0 color=blue  
2: plot fngy(ixpt1+1+ixpt2,1ysptrx,1) lpol(ixpt1+1+ixpt2,1ysptrx)-lpol0 color=red
```

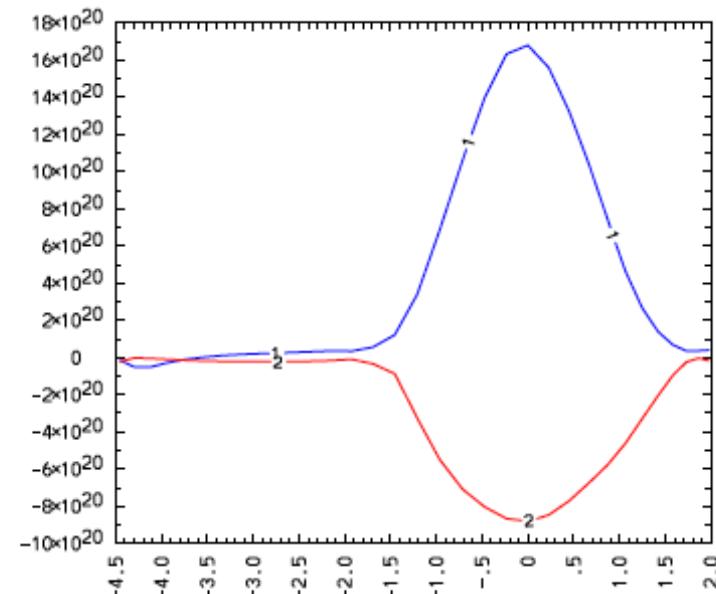
Radial deuterium fluxes, 1000 A

Core flux



```
1= plot fniy(ixpt1+1+ixpt2,0,1) lpol(ixpt1+1+ixpt2,0)-lpol0 color=blue  
2= plot fngy(ixpt1+1+ixpt2,0,1) lpol(ixpt1+1+ixpt2,0)-lpol0 color=red
```

Separatrix flux



```
1= plot fniy(ixpt1+1+ixpt2,ysptrx,1) lpol(ixpt1+1+ixpt2,ysptrx)-lpol0 color=blue  
2= plot fngy(ixpt1+1+ixpt2,ysptrx,1) lpol(ixpt1+1+ixpt2,ysptrx)-lpol0 color=red
```

Neutral gas density ...

