

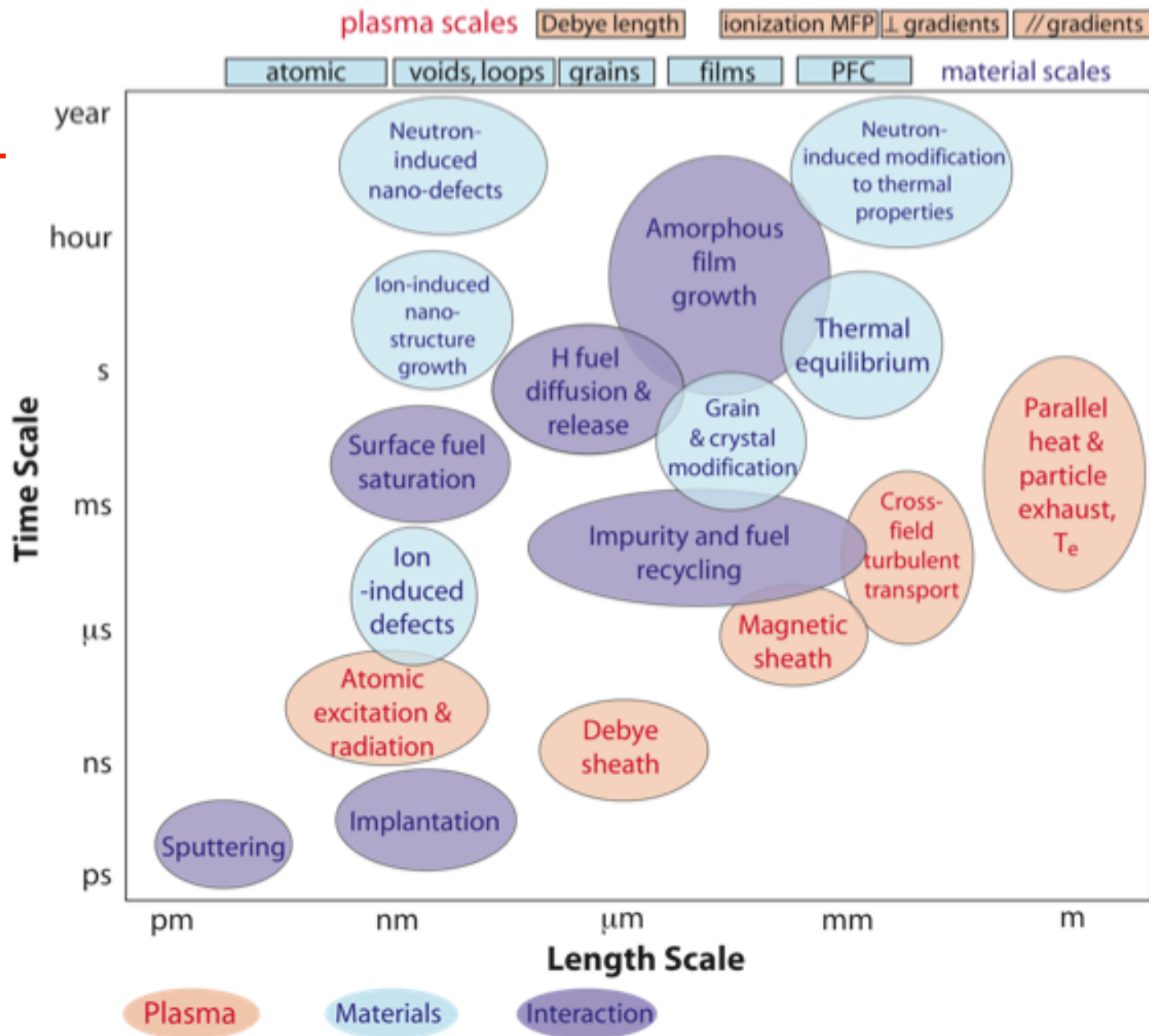
Overview of PSI Science Center Research



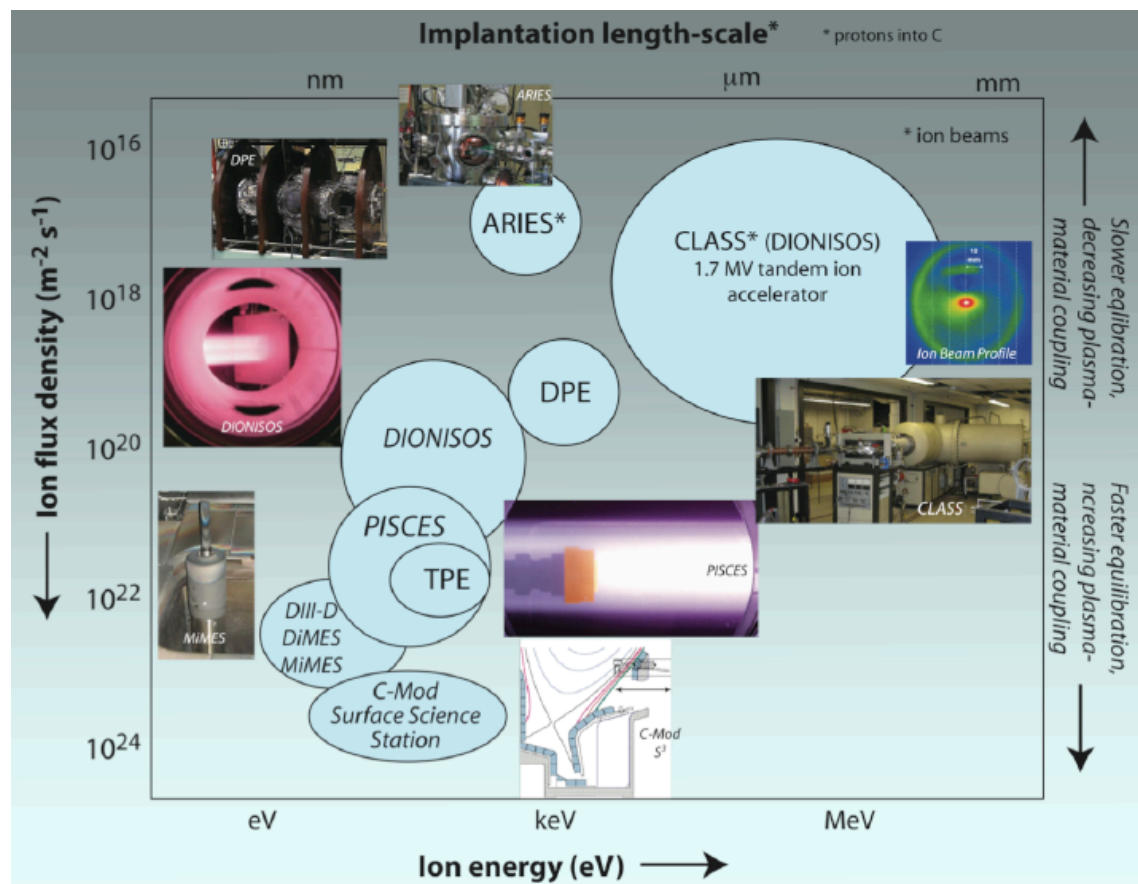
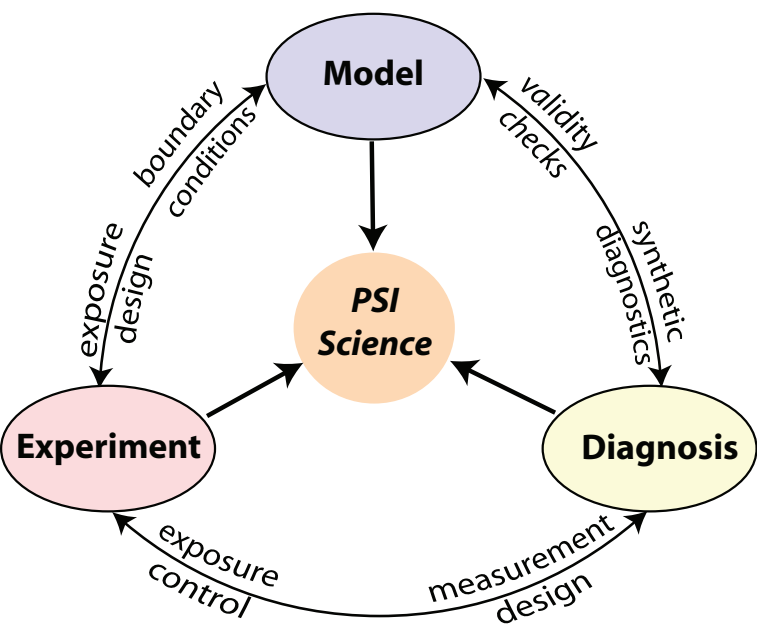
D. Whyte for PSI Science Center
PFC/MASCO Meeting PPPL June 2012



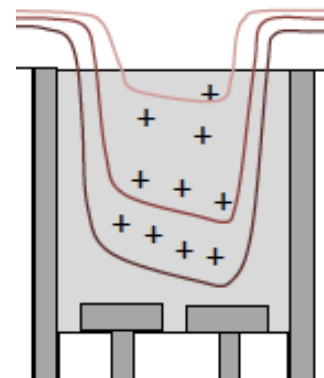
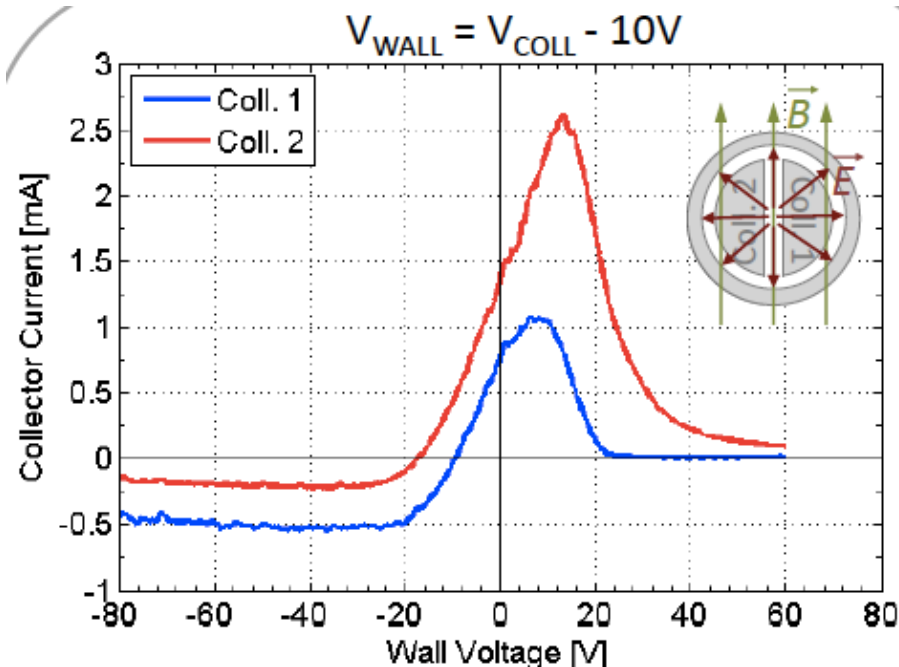
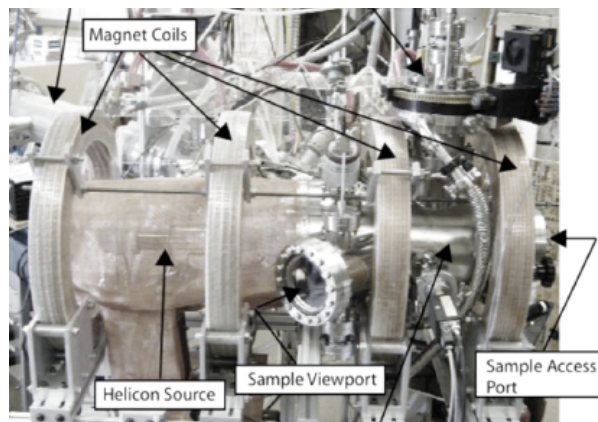
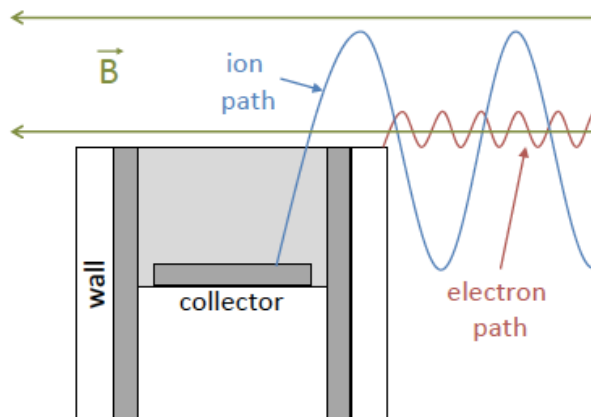
The Challenge



A holistic approach to Plasma-Surface Interaction Science



Diagnostic Development: New robust “ion-sensitive probe” for plasma potential → critical to diagnose sputtering in high density SOL

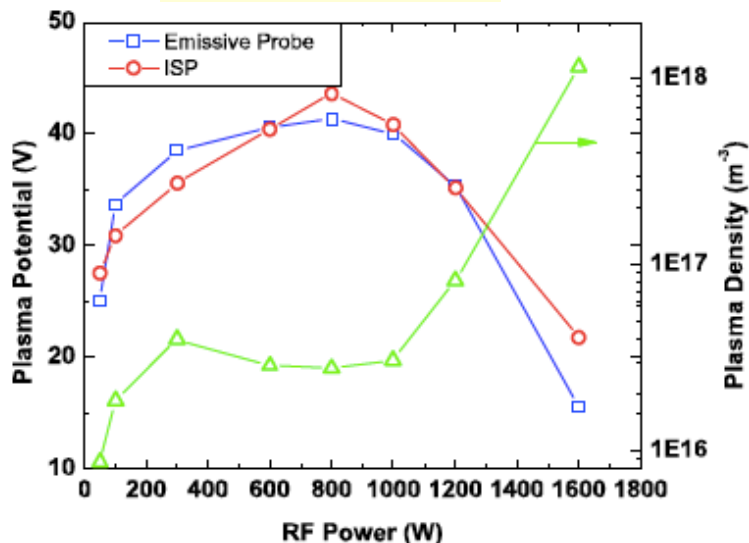


Sullivan PSI 2012

Laboratory developed ISP probe deployed in C-Mod to understand vexing RF-enhanced sheaths $\rightarrow$ high-Z sputtering

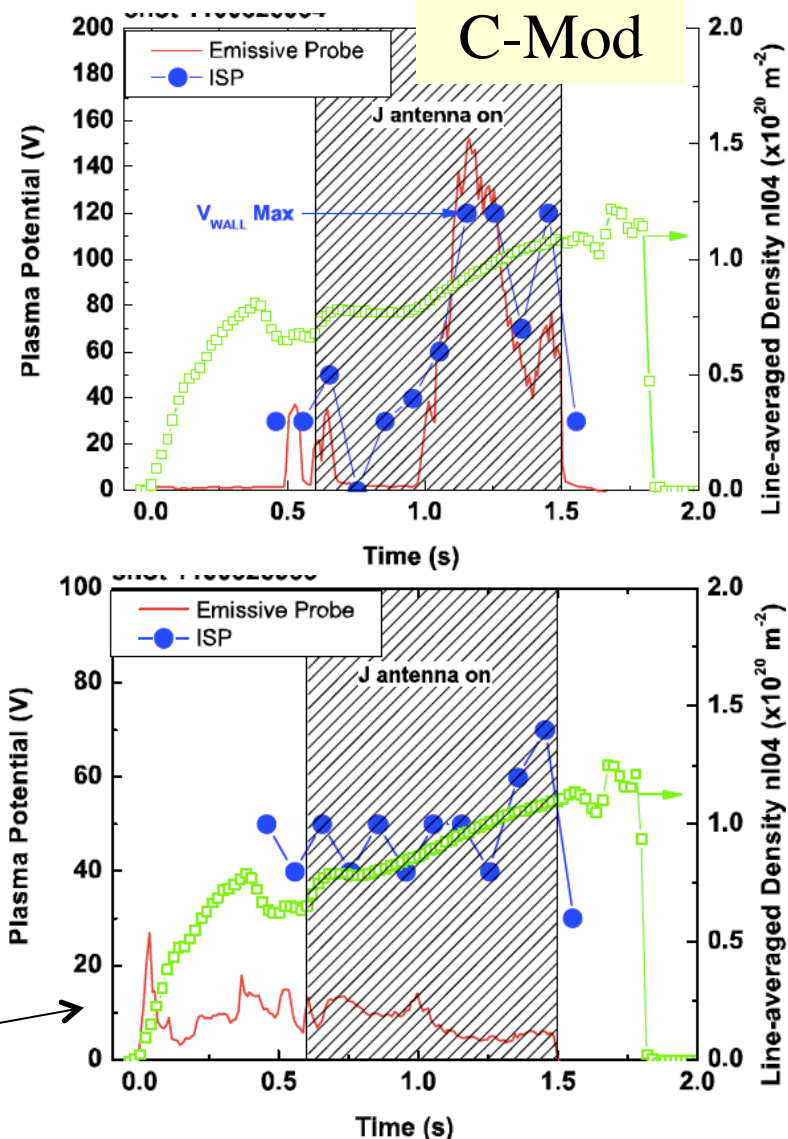


DIONISOS



Ochoukov RSI 2011

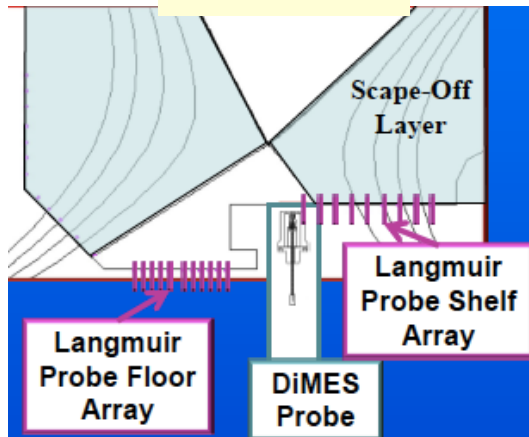
ISP functions at high density where emissive probe cannot



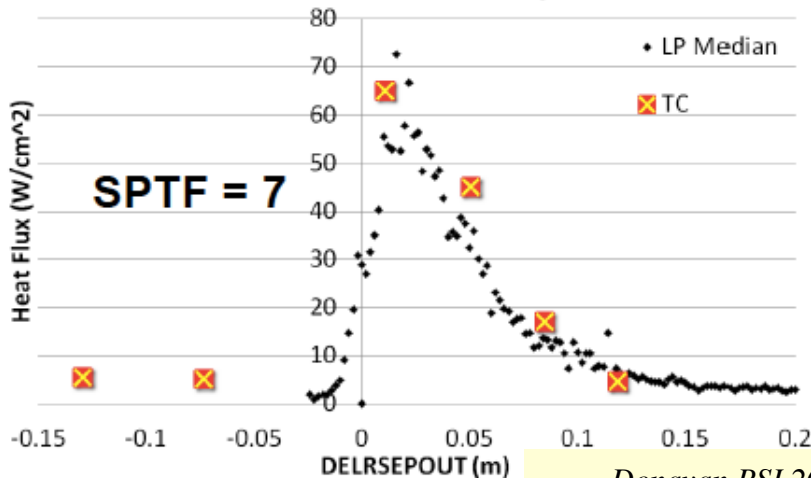
Recent significant efforts in validating sheath heat transmission at tokamak strikepoints:

Standard theory $\gamma \sim 8$ appears to be OK

DIII-D

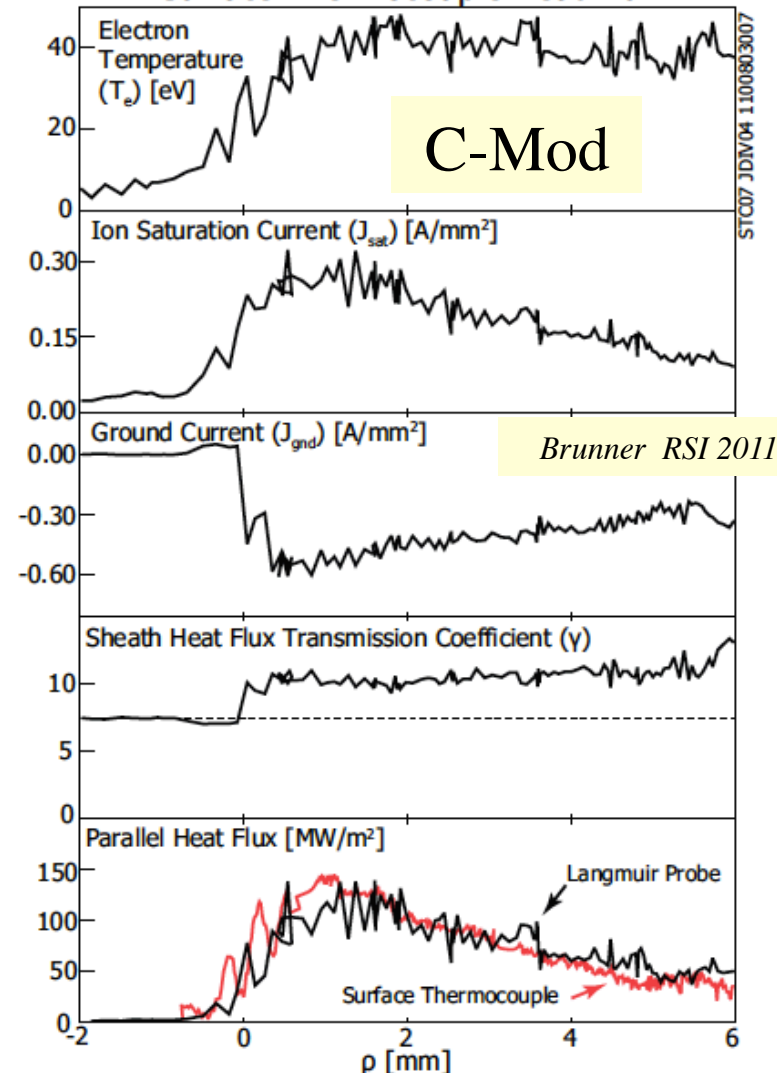


Perpendicular Heat Flux - LP Median vs Embedded Thermocouple



Donavan PSI 2012

Comparison of Langmuir Probe and Surface Thermocouple Heat Flux



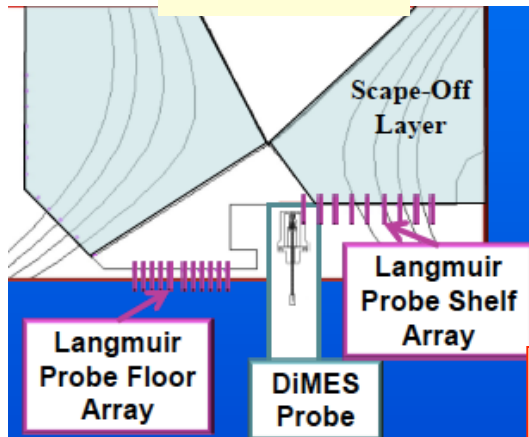
C-Mod

Brunner RSI 2011

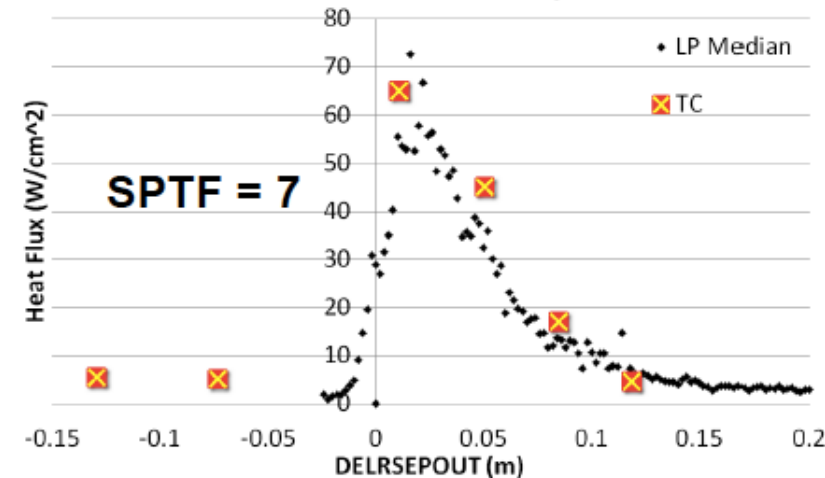
Recent significant efforts in validating sheath heat transmission at tokamak strikepoints:

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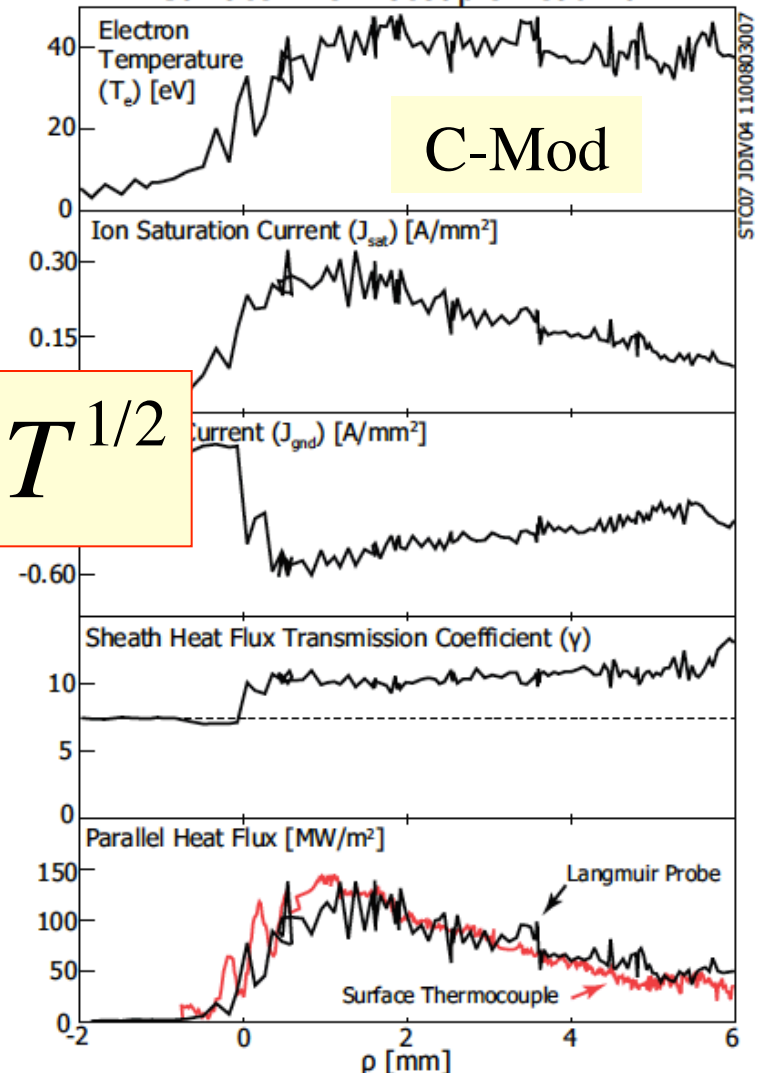


Perpendicular Heat Flux - LP Media
Embedded Thermocouple



$$q_{div} \propto \gamma p T^{1/2}$$

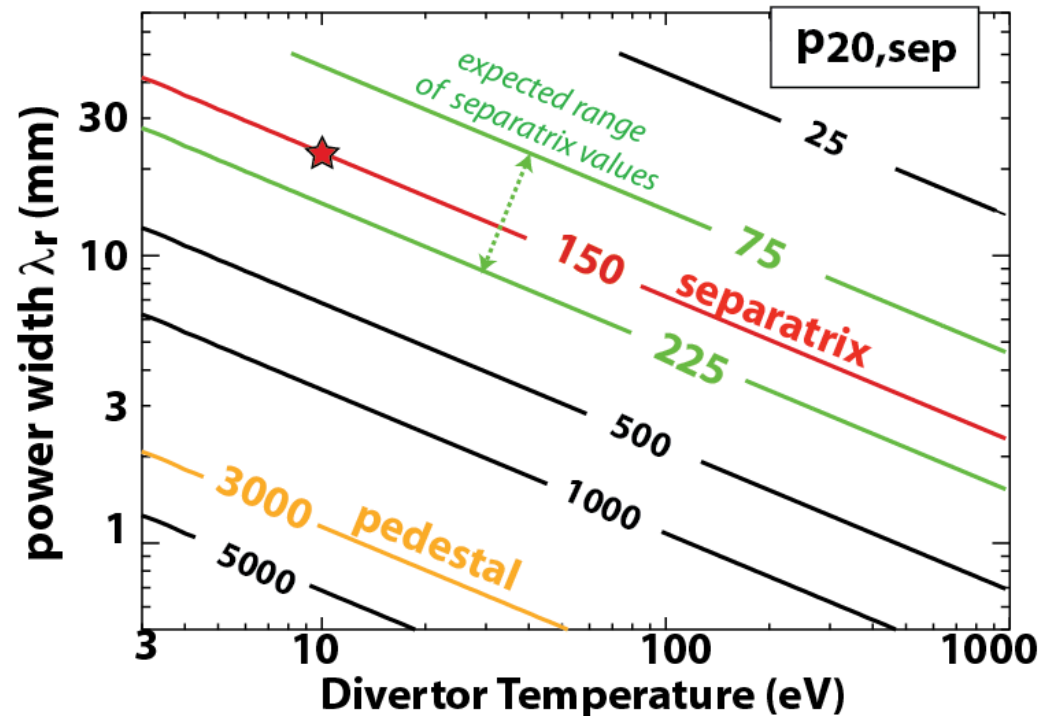
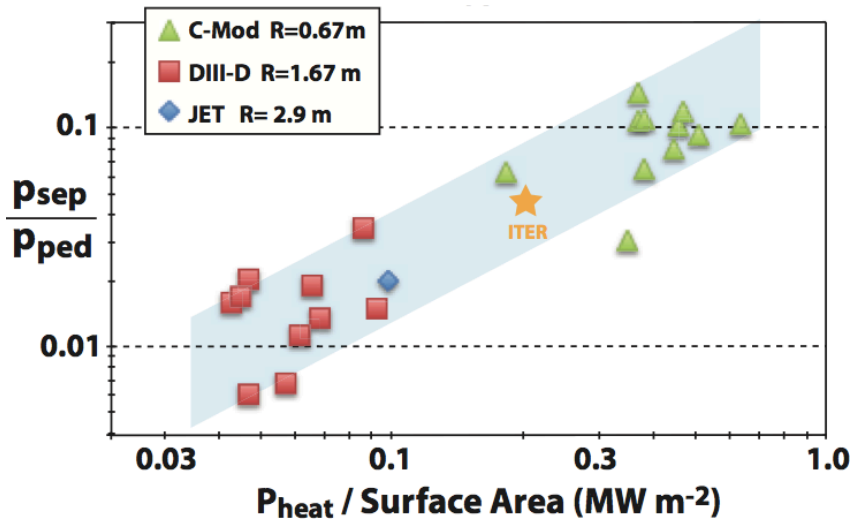
Comparison of Langmuir Probe and Surface Thermocouple Heat Flux



With confidence in sheath heat transmission, edge stability allows one to constrain allowed values of heat width in ITER

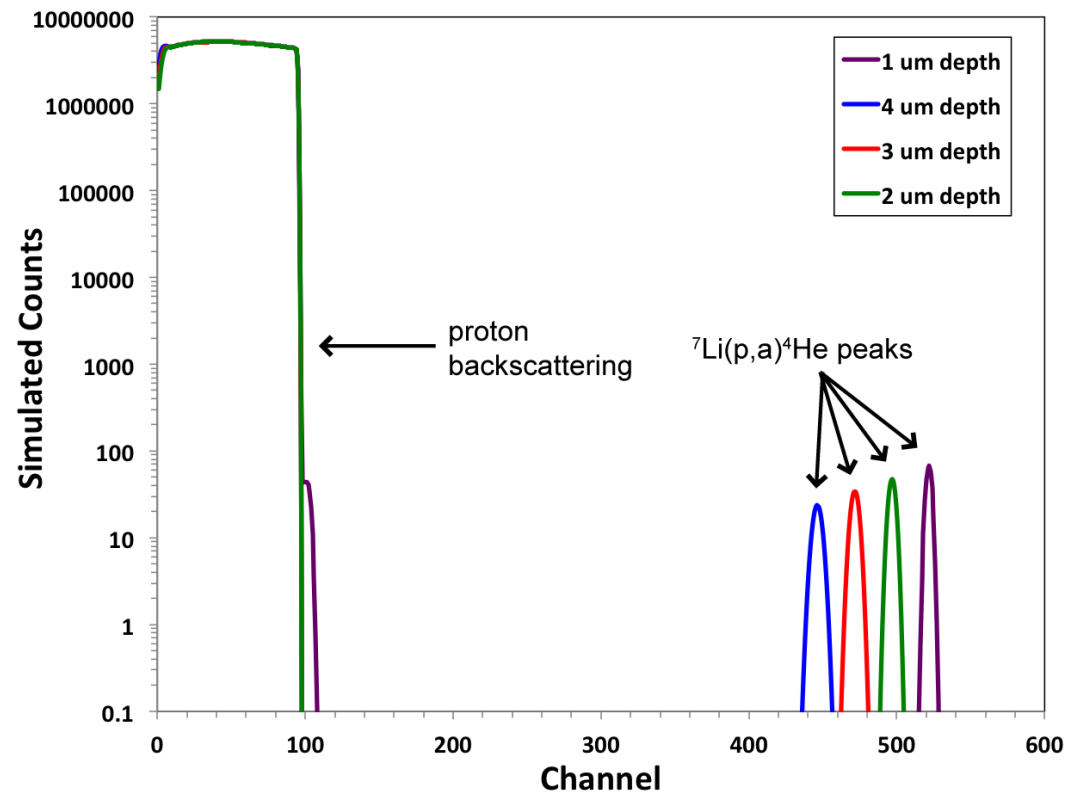
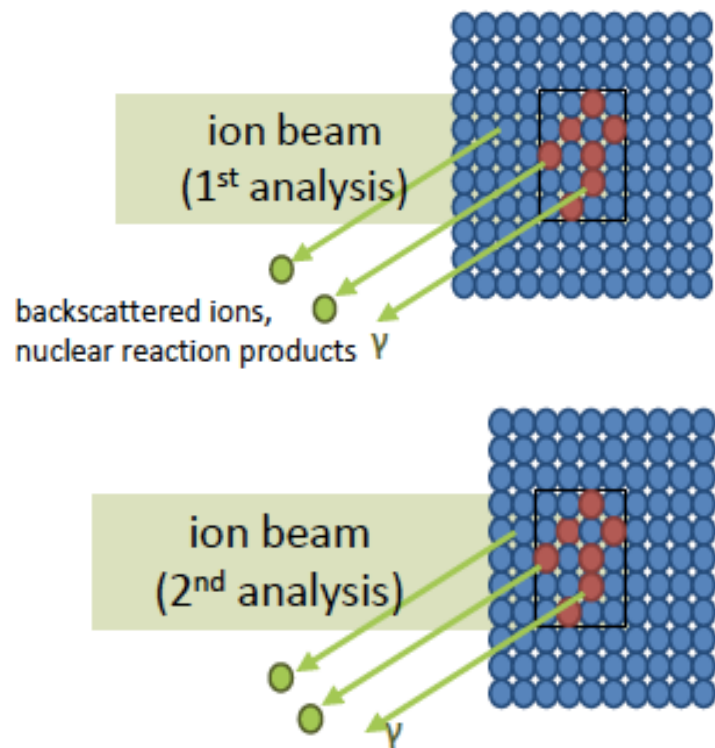
$$p_{20,sep} \approx 5\% p_{20,ped} = 150$$

$$p_{20,sep} = \frac{10.8}{T_{div}^{1/2} \lambda_r}$$

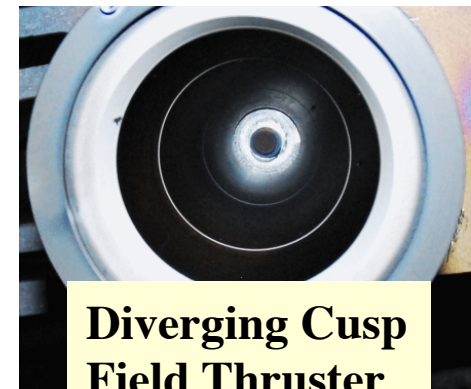
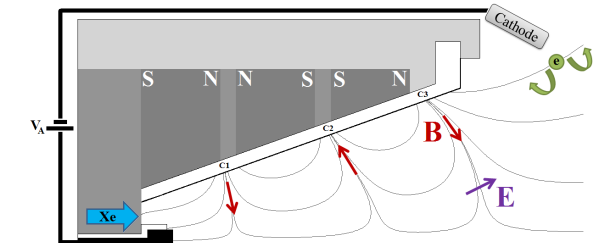
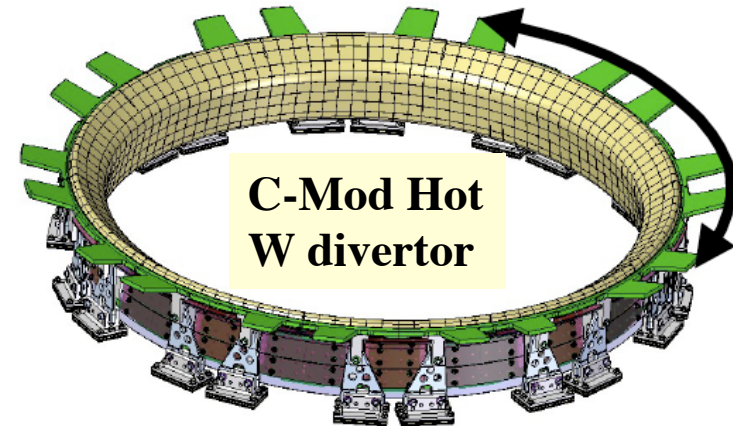
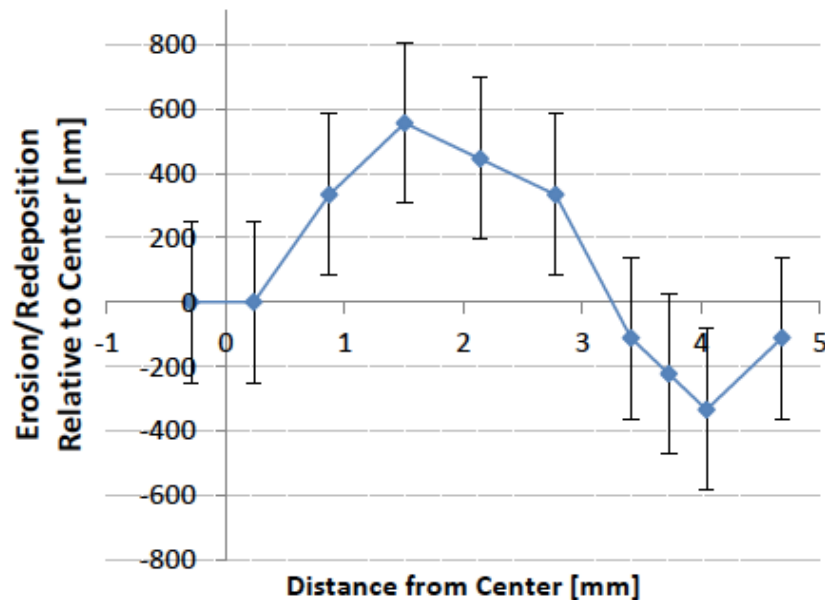
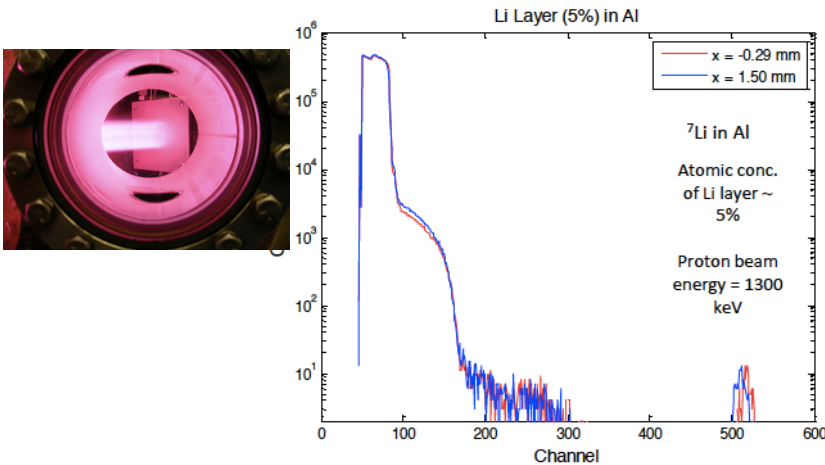


ITER $Q=10$ $P_{div}=50$ MW case

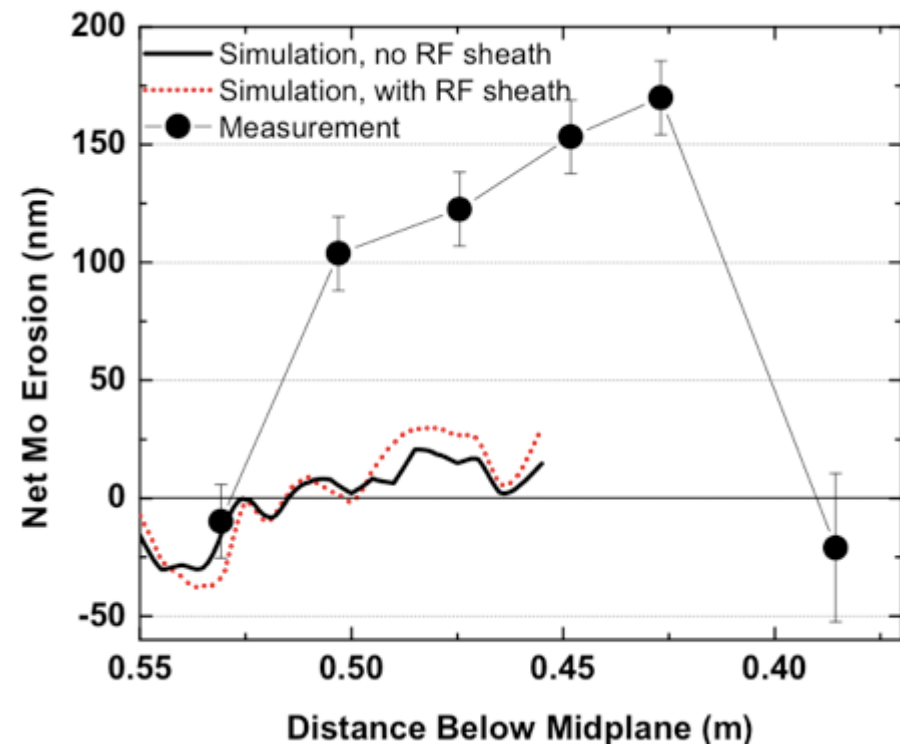
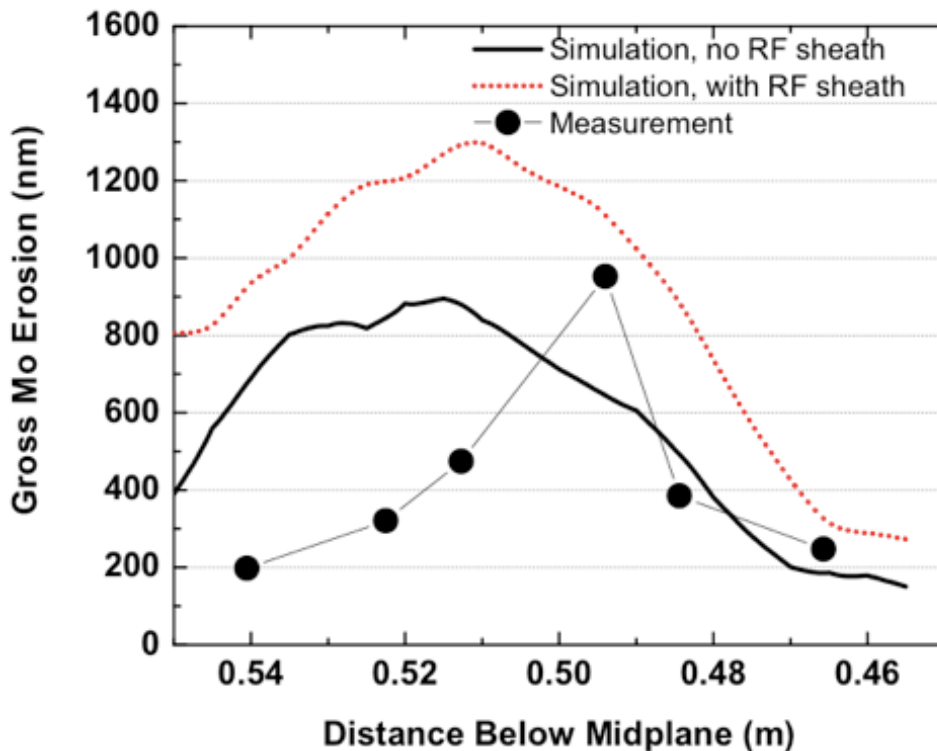
MeV Li implantation + p-Li reaction → depth-marking technique developed for high-Z and ceramics: No coating required



Depth-marking technique being used for in-situ erosion/deposition in DIONISOS → planning widescale deployment in C-Mod / EAST tungsten divertor + plasma thrusters

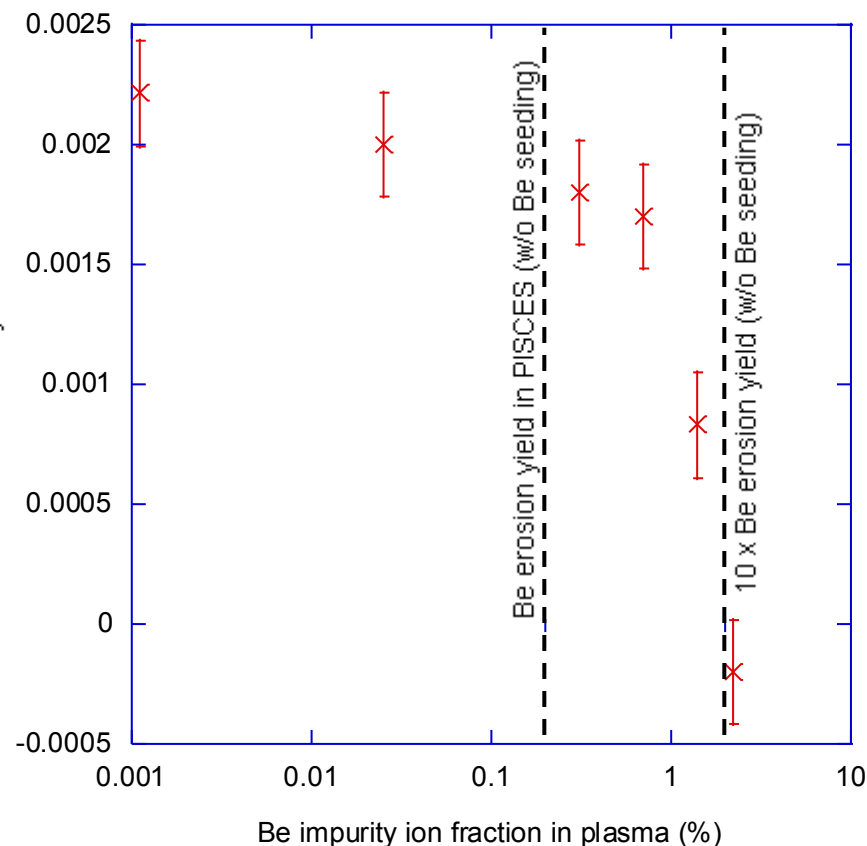
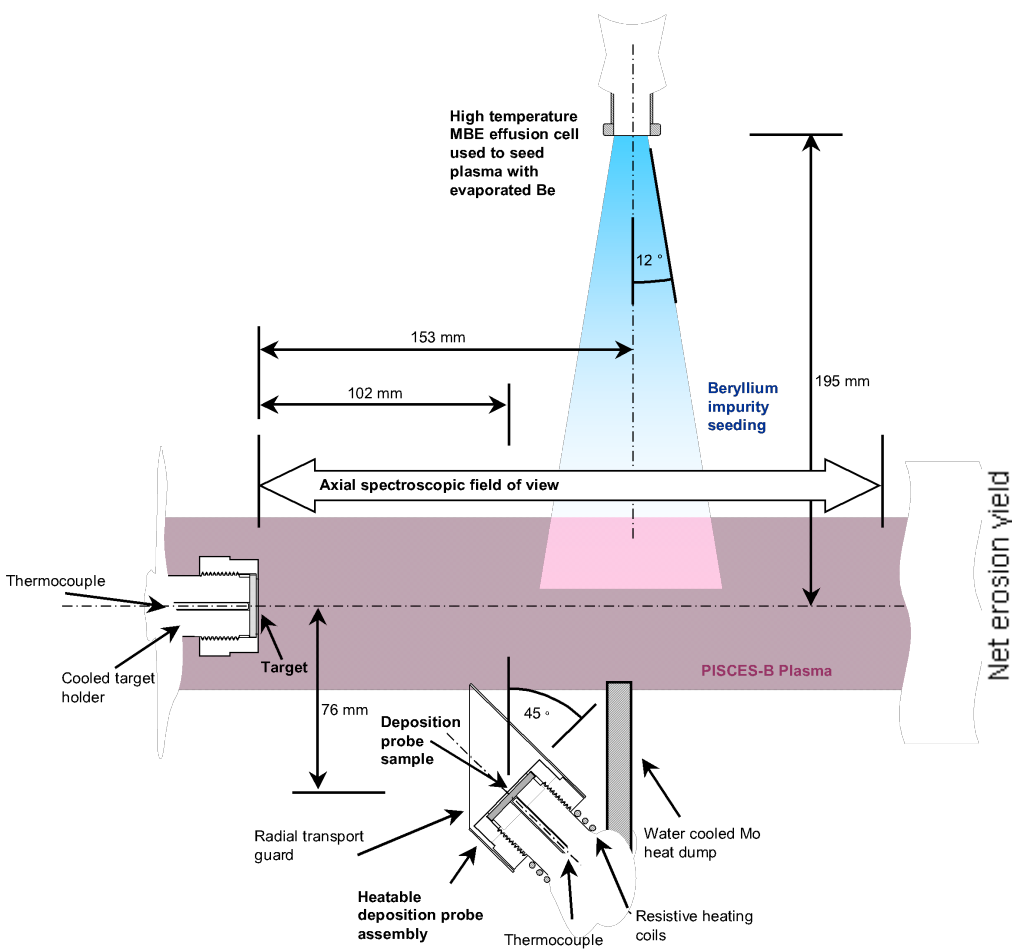


How effective is prompt redeposition at mitigating net erosion? Integrated high-Z erosion results from C-Mod *suggest* not very.



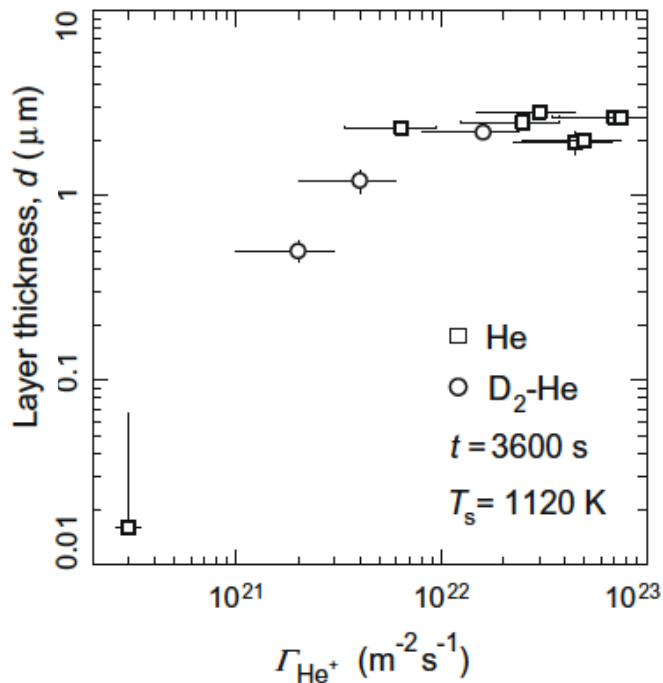
New PISCES experiments indicate our knowledge of erosion suppression by redeposition is, at best, incomplete

PISCES

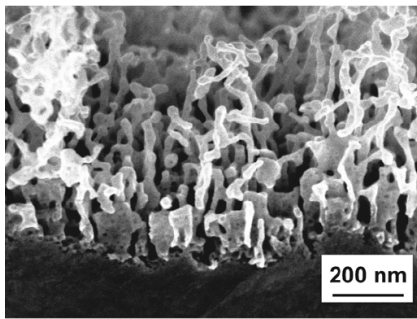


Tungsten nano-tendrils studies

Center's holistic approach well-suited to study of *W nano-tendrils*

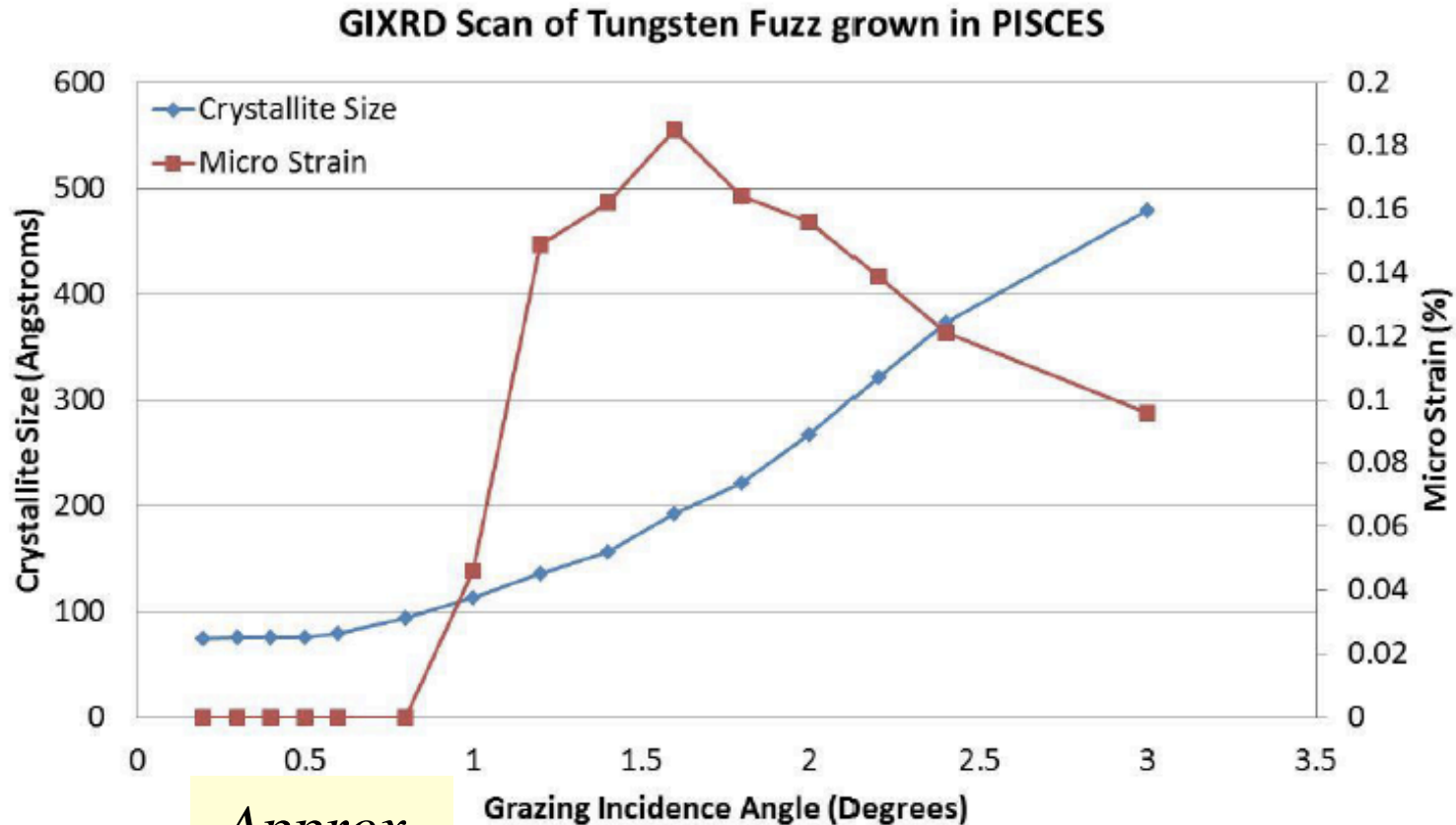


[2] M.J. Baldwin et al. J. Nucl. Mater. **390-391** (2009) 886

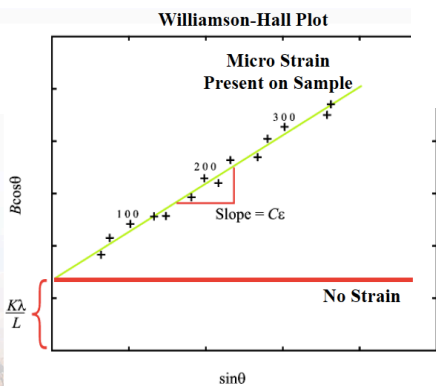


- Under what conditions will it occur?
 - PISCES key in characterizing formation rates and conditions.
 - He fluence, $1000 < T < 1800$ K
- What fundamentally causes it?
- Can it be mitigated?
- Will it occur in confinement devices?
- What will be the consequences?

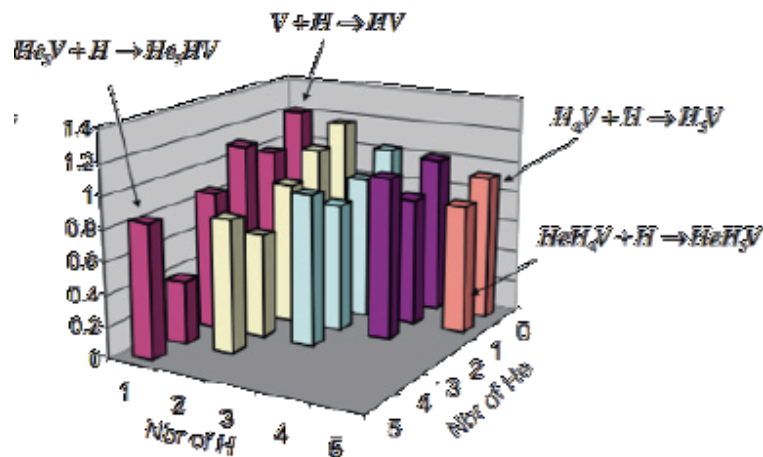
Grazing Incidence X-ray Diffraction suggest strain is relieved in fuzz → hints growth trigger is stress relief from He bubbles



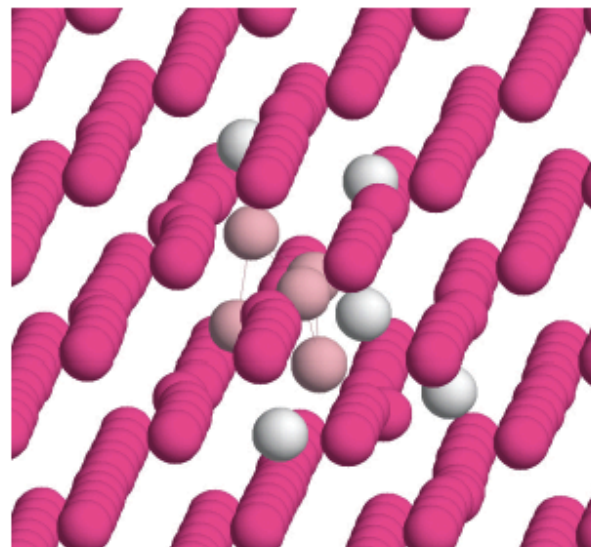
Approx.
tendrils
depth



Developing new MD simulations of complex W-He-H system



Binding energies (eV) for the reaction: $He_nH_nV + H \rightarrow He_nH_{n+1}V$.

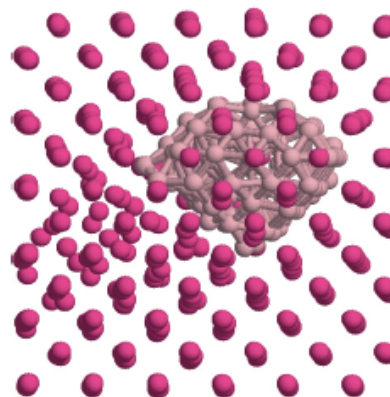


$V_1He_5H_5$

Small clusters are in symmetric structures. Harder to find ground states for large clusters, due to many metastable structures. Room for ~3 to 5 He per V for large clusters.

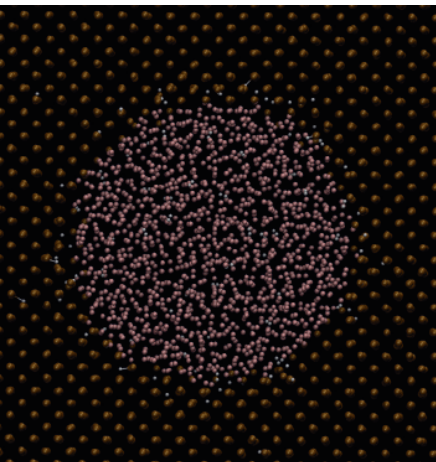


V_1He_{13}

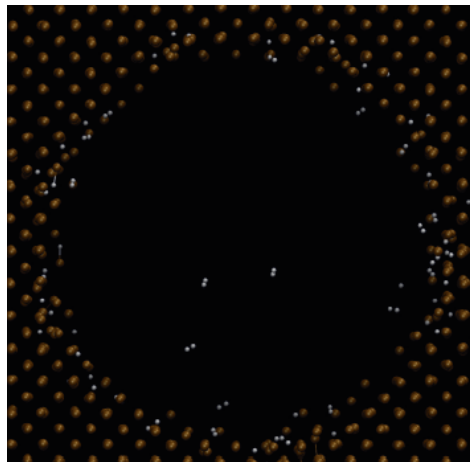


$V_{10}He_{110}$

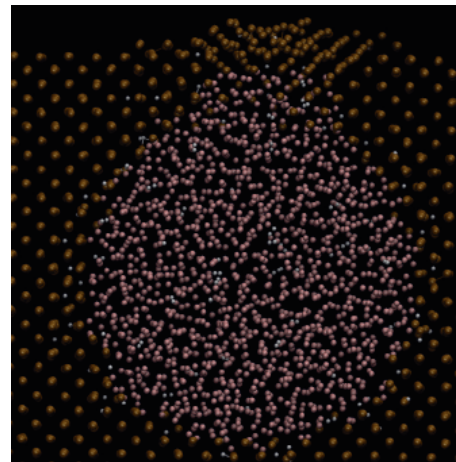
Developing new MD simulations of complex W-He-H system



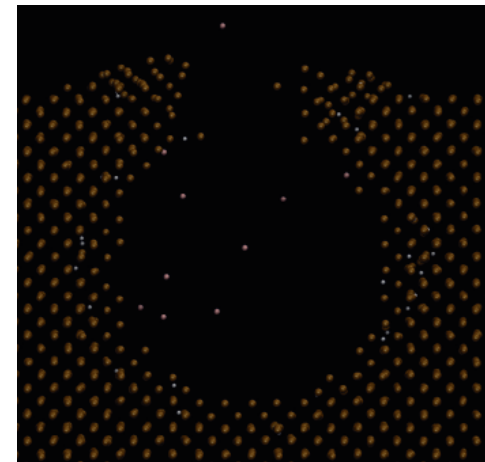
$r=2$ nm bubble, 900 K
 $1 \cdot n_{\text{vac}}$ He & $0.1 \cdot n_{\text{vac}}$ H



$r=2$ nm bubble, 900 K, He not shown
 $2 \cdot n_{\text{vac}}$ He & $0.1 \cdot n_{\text{vac}}$ H



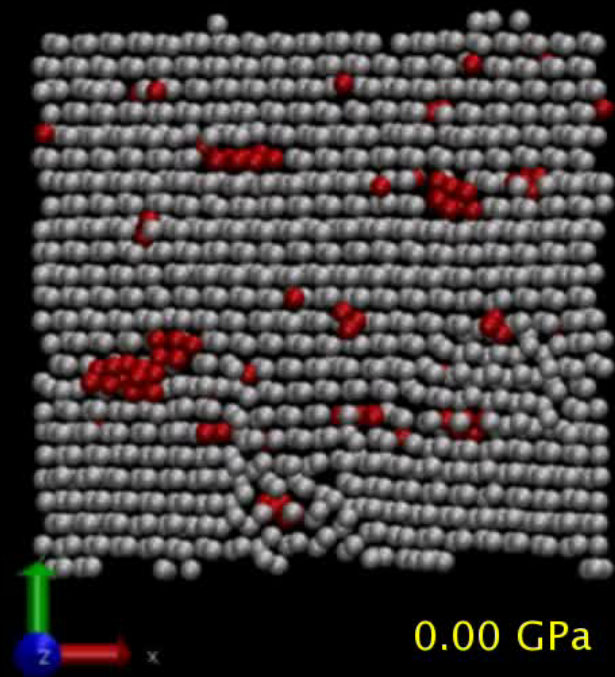
$r=2$ nm bubble near surface, 900 K



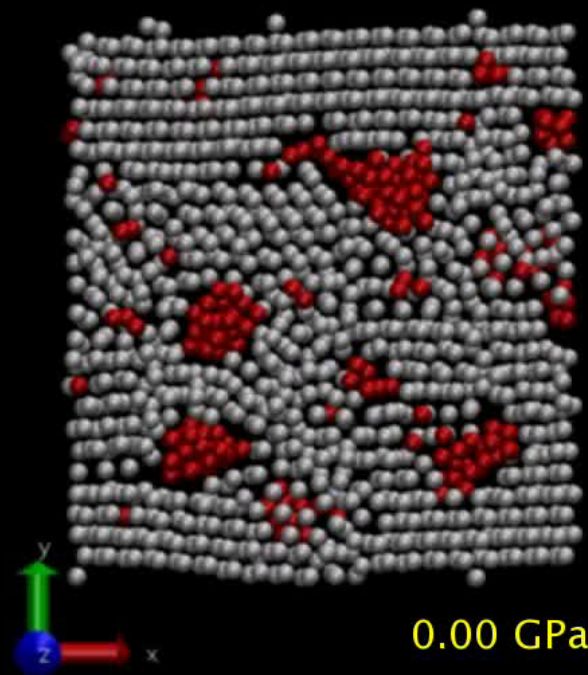
$r=2$ nm burst bubble
1% He & 35% H retained

MD simulation with applied shear stress: tungsten “movement” depends on He %

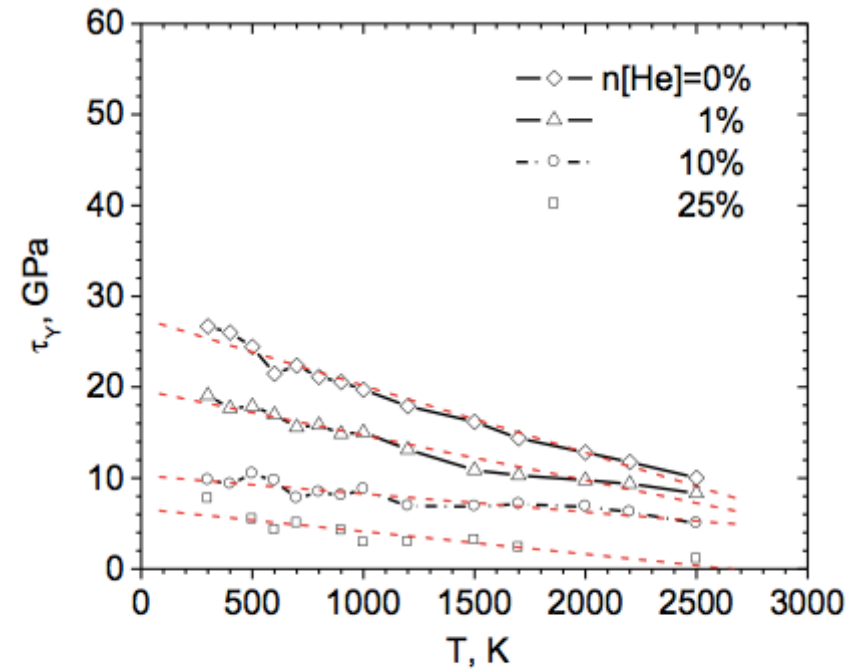
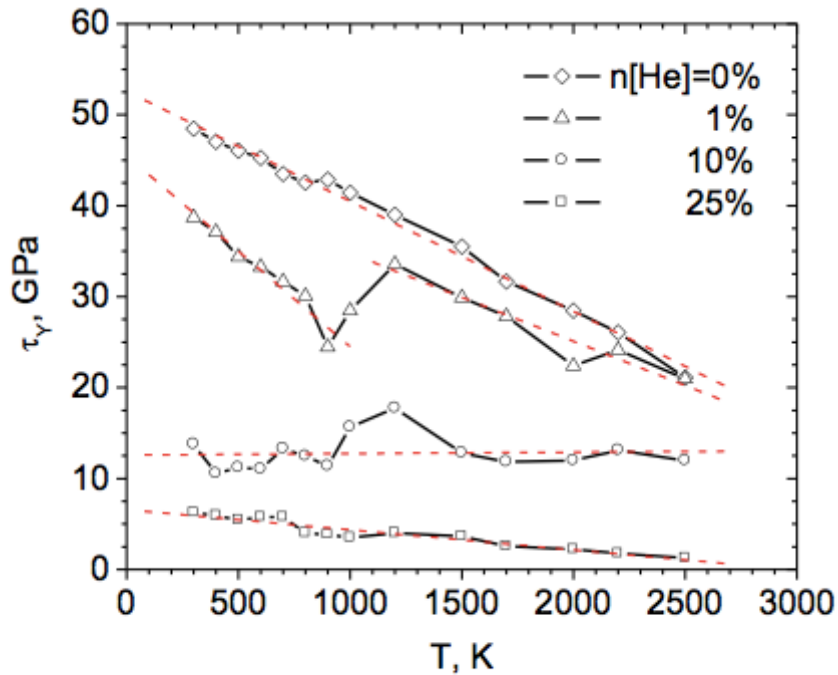
W{110} n[He]=10 at.% T=1000K



W{110} n[He]=25 at.% T=1000K



MD simulations indicate that ~10% He concentrations strongly affect effective yield strength of tungsten

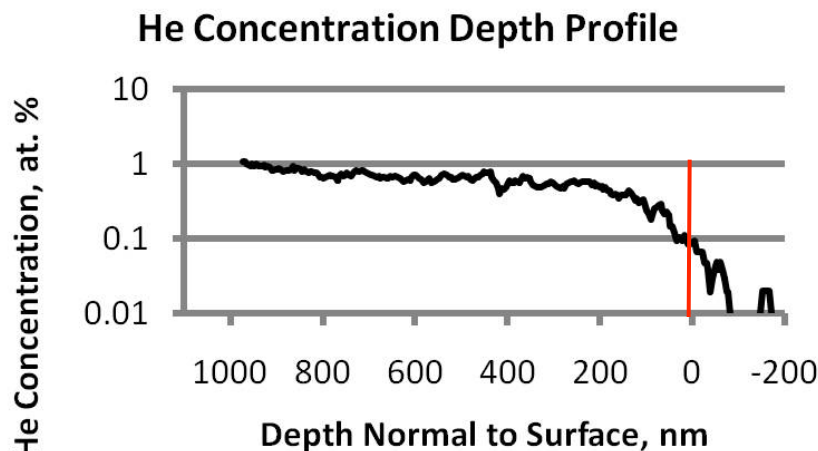
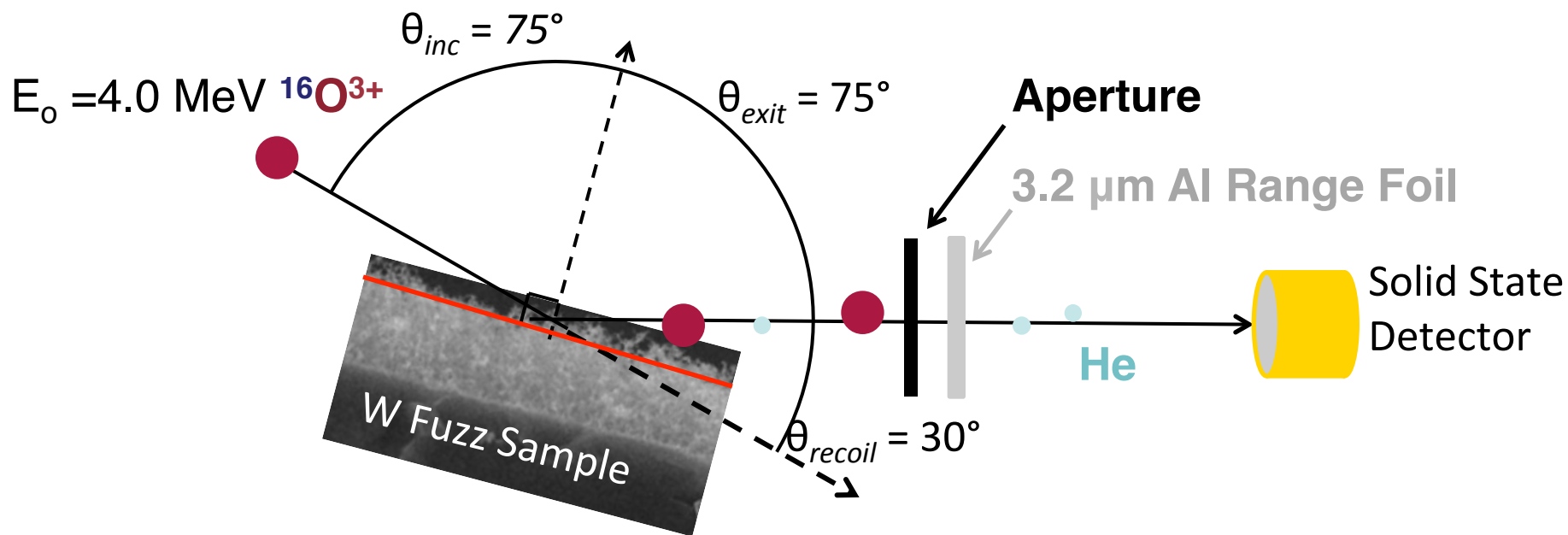


a)  

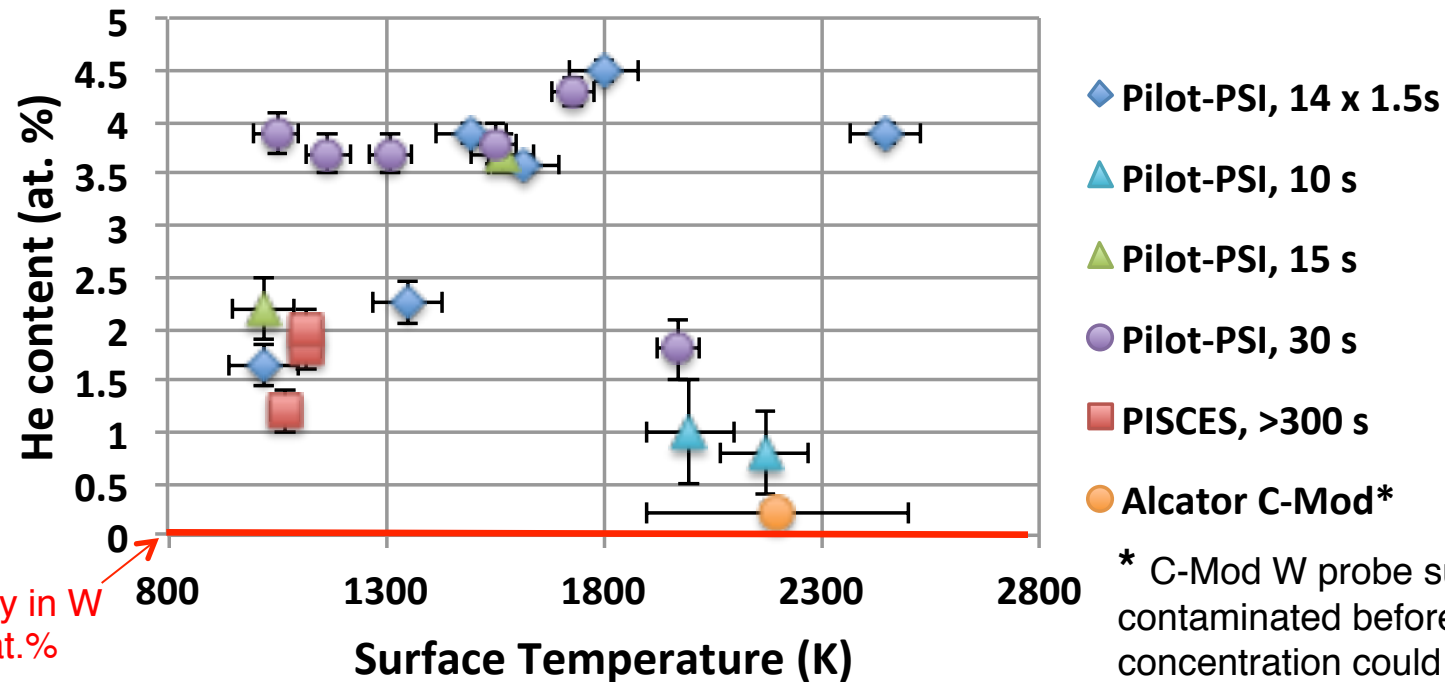
b)  

Fig.1 Simulated dependencies of the shear yield strength of tungsten with different initial atomic concentrations of dissolved helium for the cases of shear stress applied in a) $\{010\}\langle 100 \rangle$ and b) $\{110\}\langle \bar{1}11 \rangle$ directions. 

Diagnostic: Heavy ion elastic recoil detection allows us to measure the He depth profile in the surface layer/fuzz



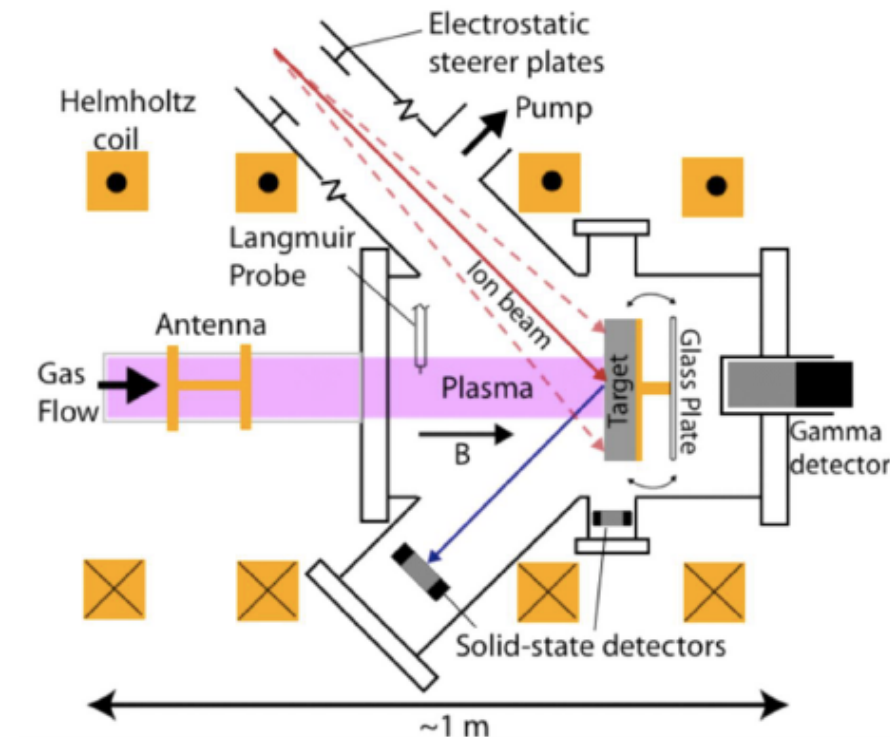
He content in tungsten targets exposed in Pilot-PSI and PISCES indicates a lack of high He pressures in voids



- Concentration is well above natural solubility, but below what might be expected for ~GPa He pressures in all voids.
- No clear trend vs. surface temperature, despite varying void sizes.

We are starting dynamic measurement of He content (+ W migration with marker) in DIONISOS

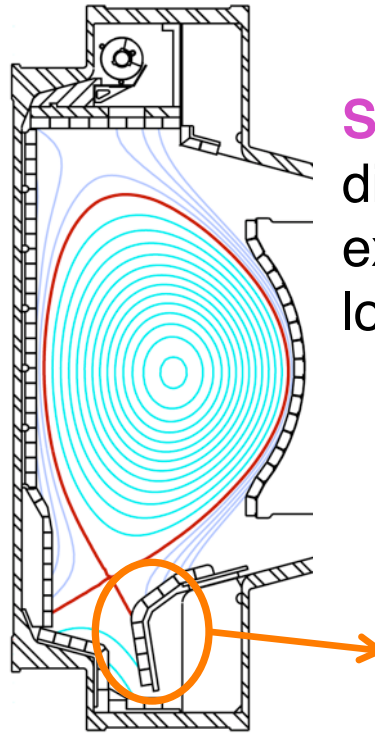
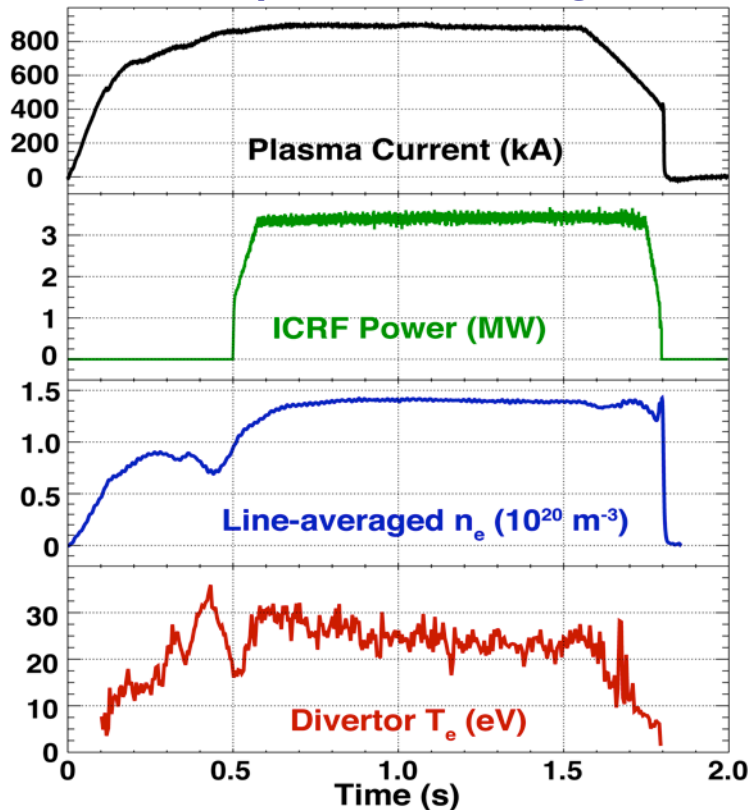
- Is the He content higher when the plasma is present?
- How does the He migrated versus depth at the beginning of growth and as growth saturates?
- In what manner does the tungsten grow from the sample?
 - Grain orientation /w single crystal W?



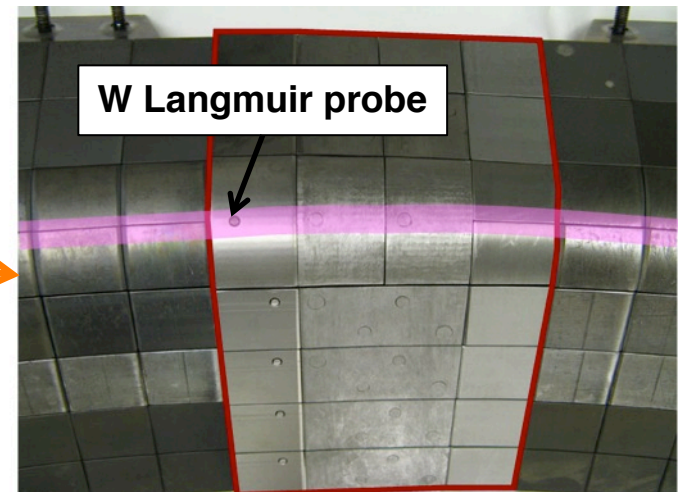
Alcator C-Mod helium plasmas produced necessary plasma conditions for fuzz growth at the outer strike point



He plasma discharges



Strike point run above vertical divertor face to reduce flux expansion allowing for higher local surface temperatures.



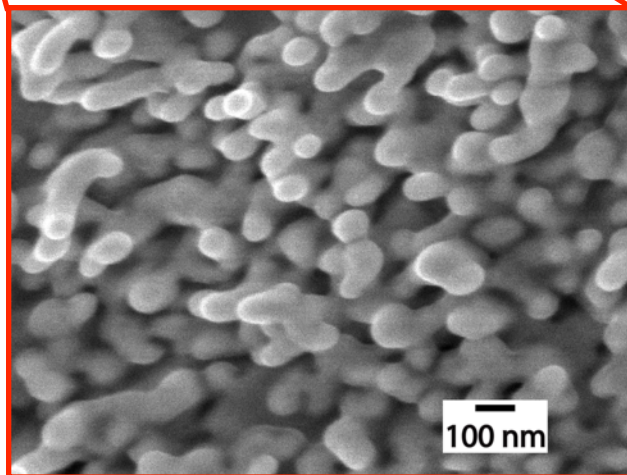
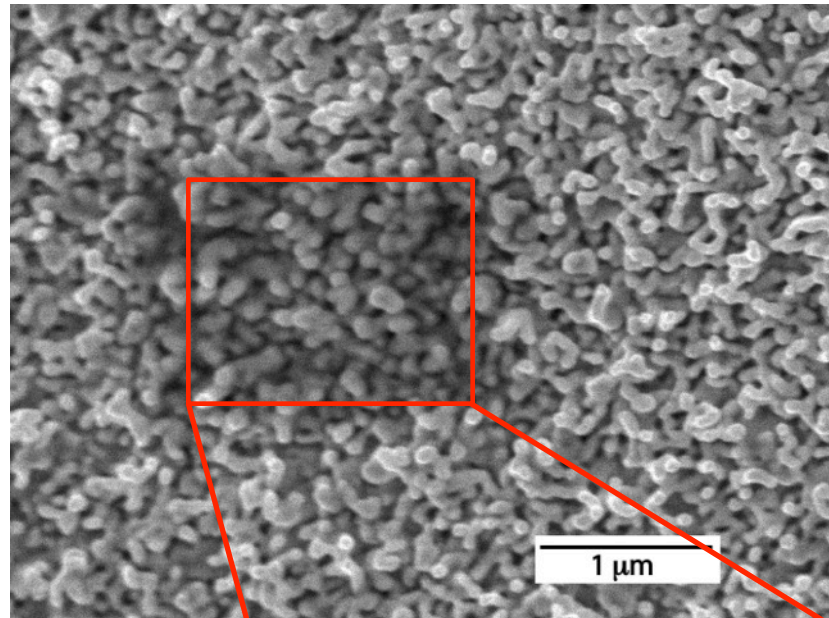
Heat flux instrumented tiles

- 14 repeated L-mode discharges
- $T_{e,\text{divertor}}$ 20-25 eV, $q_{\parallel} > 0.2 \text{ GW/m}^2$

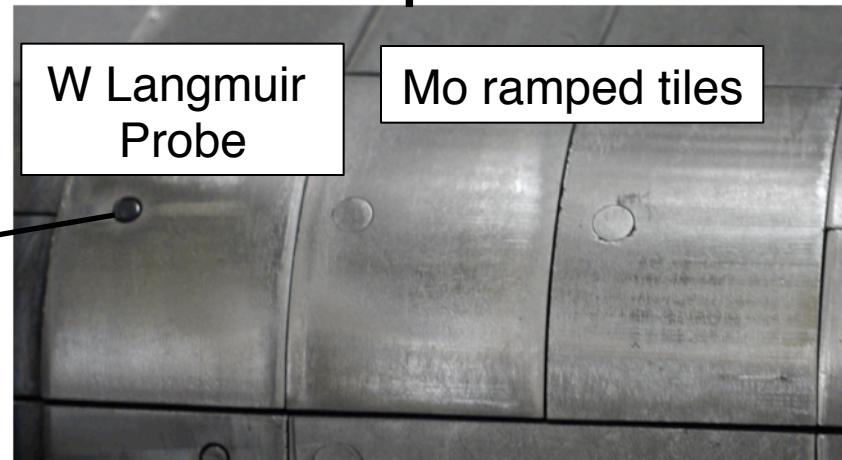
→ ~13 s of total exposure at appropriate growth conditions

Nano-tendrils are fully formed on tungsten divertor probe exposed to heat fluxes of 30-40 MW/m²

Will tendrils grow in tokamak divertors? YES

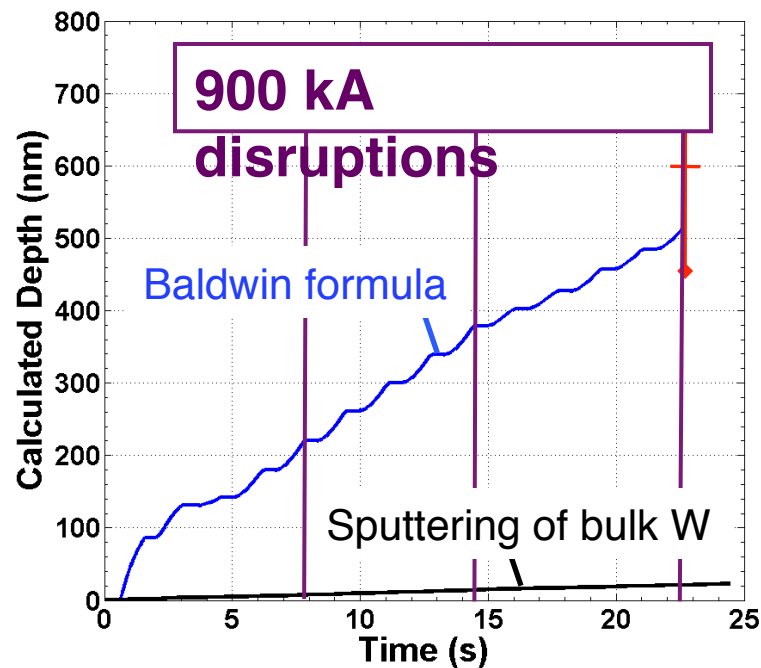


After exposure



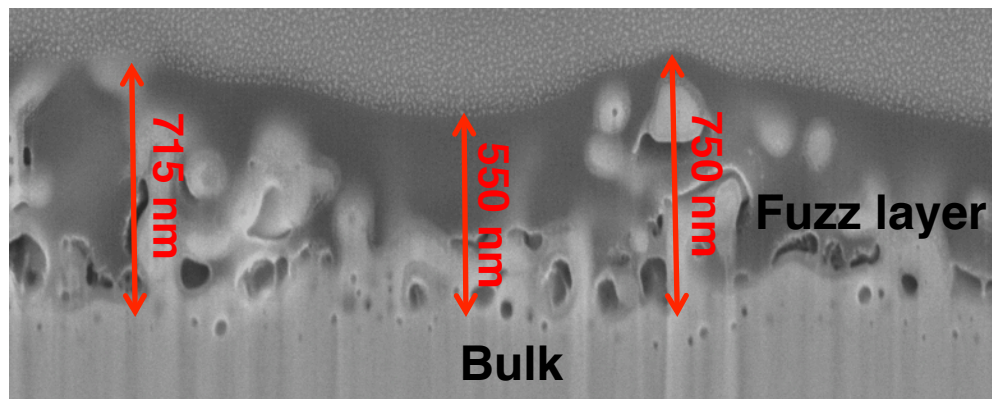
Thickness of individual tendril is ~100 nm.

The PISCES empirical growth rate appears applicable to C-Mod results



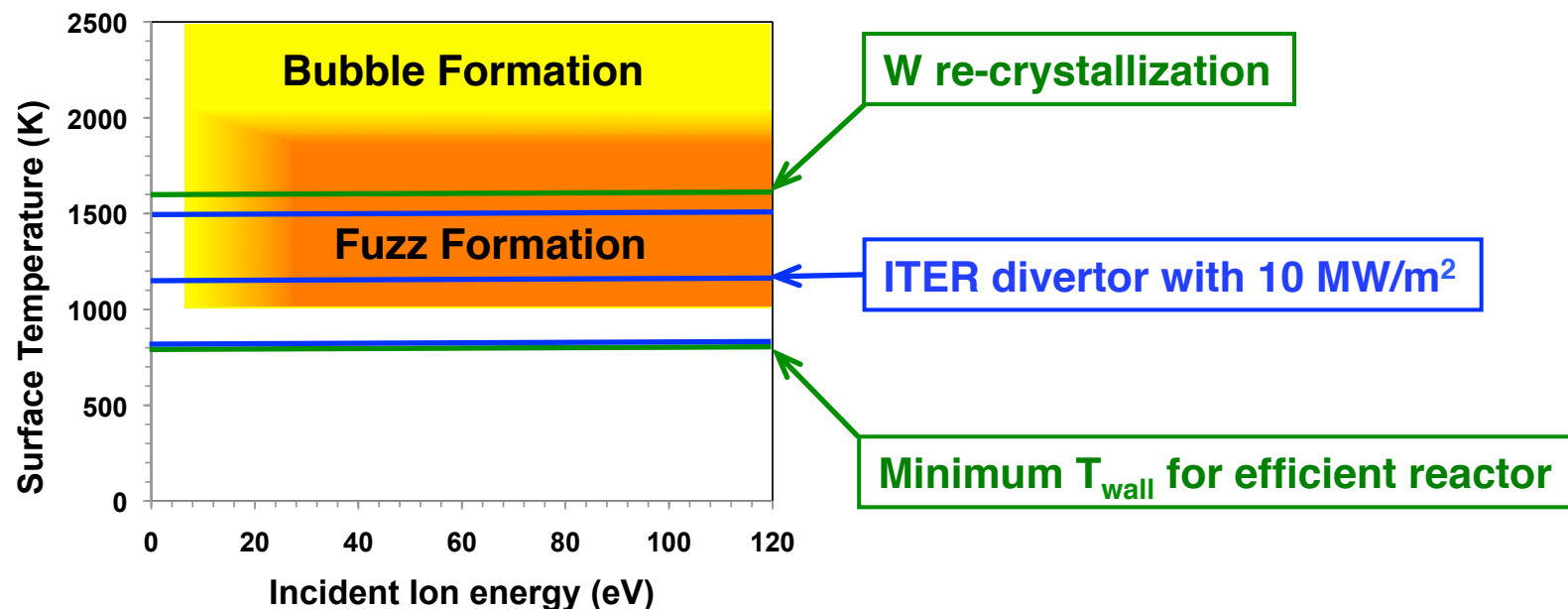
- Growth is estimated through $t^{1/2}$ -dependence:

$$\text{layer depth} = \delta \times G(T_{\text{surf}}) \times t^{1/2}$$
 where $G \propto \exp(-E_{\text{act}}/kT_{\text{surf}})$, $E_{\text{act}} = 0.71 \text{ eV}$
 [3] M.J. Baldwin, R.P. Doerner, Nucl. Fusion 48 (2008) 035001
- Calculated cumulative layer depth of **~520 nm** for W probe
- Sputtering only a small contribution** in W case (**~35 nm** of calculated bulk W gross sputtering)



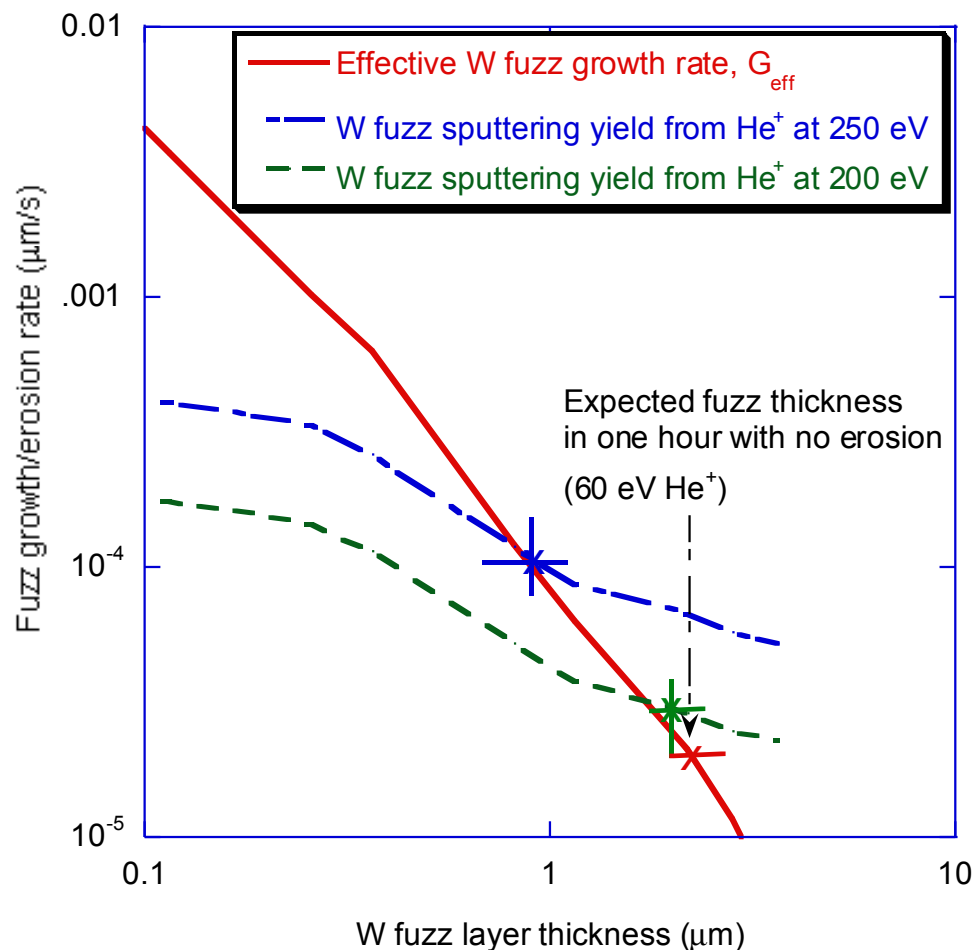
- The **measured fuzz layer thickness** was **$600 \pm 150 \text{ nm}$** from FIB cross-sectioning.

Unless we can find a mitigation method, nanotendrils W surfaces seem nearly inevitable in ITER & reactor



- Tendrils appear right in the “sweet-spot” T for tungsten.
- What will be consequences?
 - Unclear if positive or negative...but it's fairly clear our standard surface picture is out the window

PISCES exploring balance between growth and erosion → understanding equilibrium

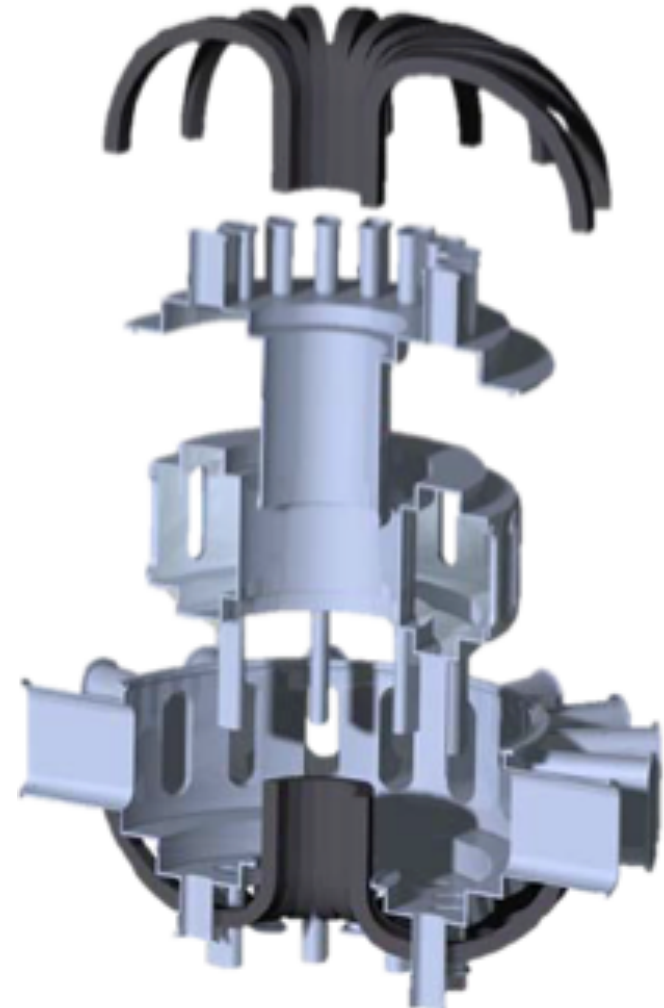


- Use measured erosion yield for He on W (i.e. net yield)
- Erosion from sputtering balances fuzz growth due to $\sqrt{t}$ dependence
- But the point of using W in divertor is to turn off sputtering? (basically required for reactor)

Educating a new generation of PSI-aware fusion scientists: e.g. conceptual design of 24/7 tokamak for PSI research



- $R \sim 1.2$ m, $B \sim 6$ T, $P_{CD} \sim 20$ MW
- 800 C vacuum vessel / wall
- Demountable superconducting coils
- Modular replacement of VV and PFCS.
- Used dimensionless similarity arguments to match ~ 15 key figures of merit for a reactor
 - SOL + divertor plasma
 - PMI, erosion, deposition
 - PFC material response.
- Four of five papers in F.E.D. [2012] special issue first-authored by students



Exploded view of Vulcan tokamak

The idea that PSI is an important *science* area is starting to permeate through the fusion community



- **There is a growing sense of appreciation in the confinement community about moving → reactor-relevant tests in confinement devices**
 - **Proper temperature! [FESAC Material report]**
 - **Center participants at APS-DPP 2012 (review + invited)**
- **We hope to continue growing this trend through our Center activities although we do face challenges**
 - **Possible C-Mod shutdown (on advent of heated W divertor)**
 - **NSTX-Upgrade delayed**
 - **DIII-D carbon environment.**
 - **International?**
- **Experience so far indicates, we believe, the importance of both using a holistic approach and engaging the confinement community and devices to advance PSI science.**